

# **BENCHMARK SPECIFICATIONS FOR ZPPR-9, JOYO and JAEA FBR**

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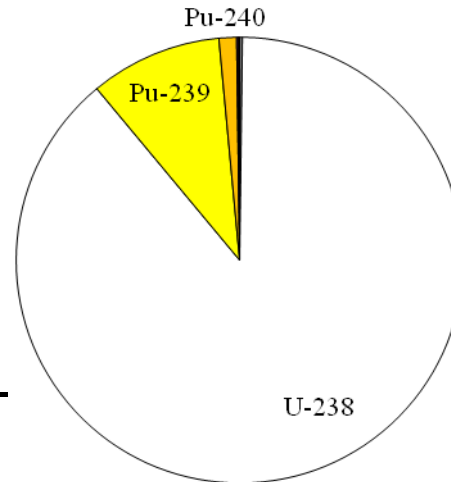


# Brief Description of ZPPR-9 and Joyo MK-I

## ZPPR-9

IRPhEP ID: "ZPPR-LMFR-EXP-002"

- Date: May - September 1978
- Size: 4600-liter
- Shape: Cylindrical and clean
- Mock-up type: Conventional mixed-oxide-fueled two-zone liquid metal fast breeder reactors of about 650 MWe



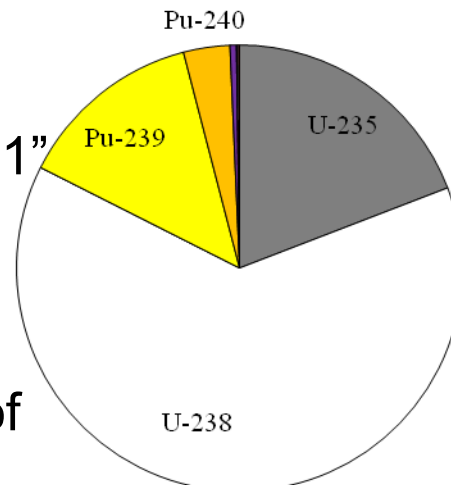
Fuel composition  
(Inner core region)



## Joyo MK-I

IRPhEP ID: "JOYO-LMFR-RESR-001"

- First criticality: April 24, 1977
- Fuel: Plutonium-uranium mixed oxide (MOX)
- Surrounding: Radial/axial blanket of depleted uranium oxide
- Maximum thermal power: 75 MWt



Fuel composition  
(Driver region)



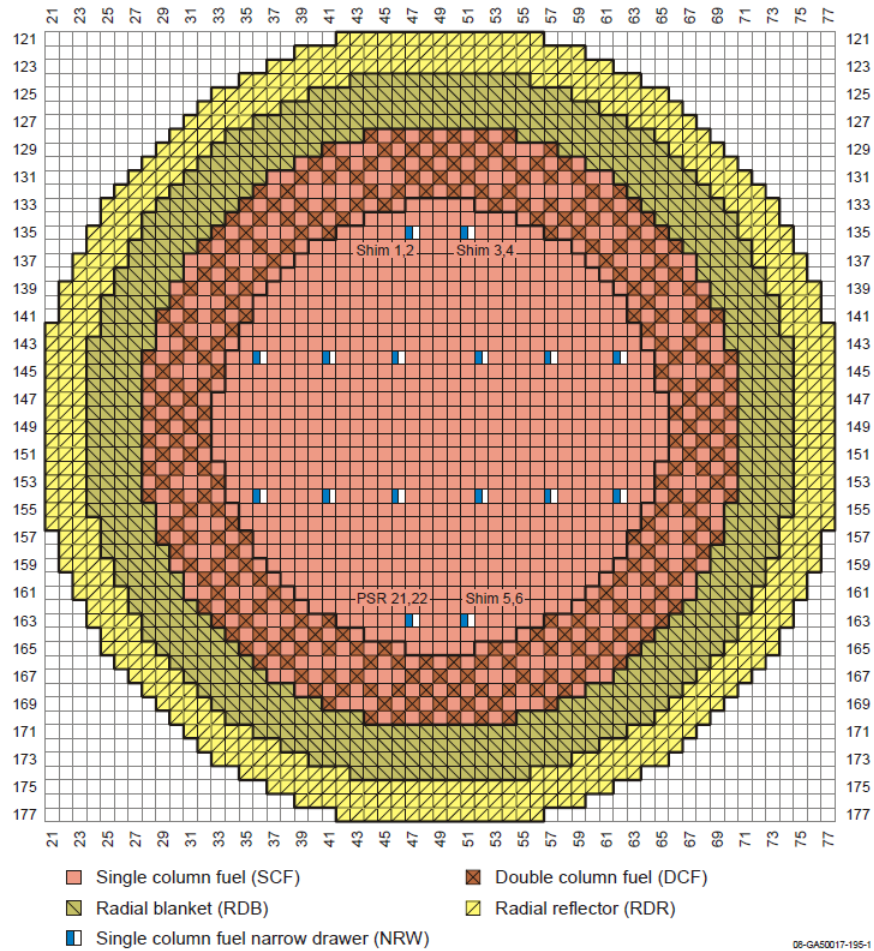


Fig. 1.8 ZPPR-9 Reference Configuration (Half 1)

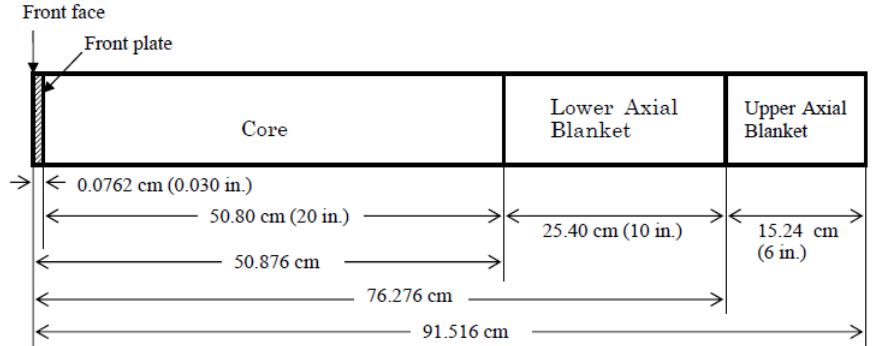


Fig. 1.9 The Relation between Drawer Configuration and Core Length

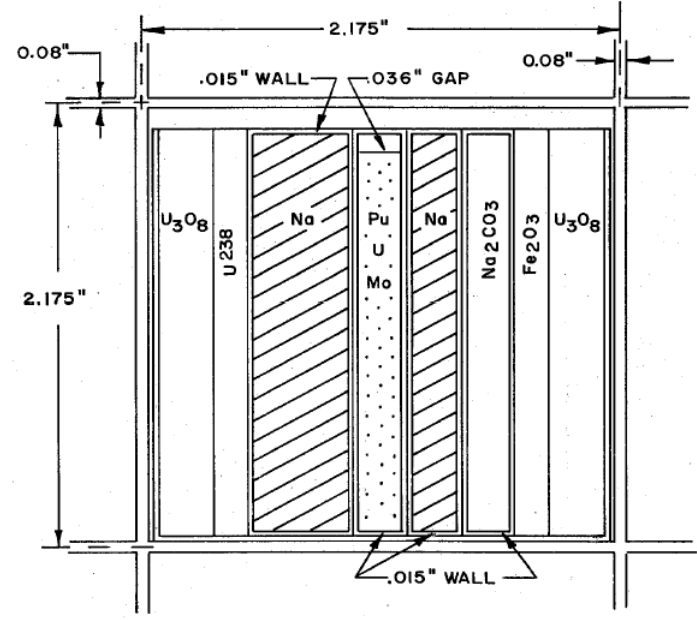


Fig. 1.10 Typical Cross Section of a Drawer in a Matrix Tube (Ref.: "The JUPITER Program: ANL Analysis of ZPPR-9", PNC SA385 85-01, Argonne National Laboratory (Sep. 1980).)

# Fig. Configuration of ZPPR-9

## (Radial layout, axial layout, cross-section of a drawer)



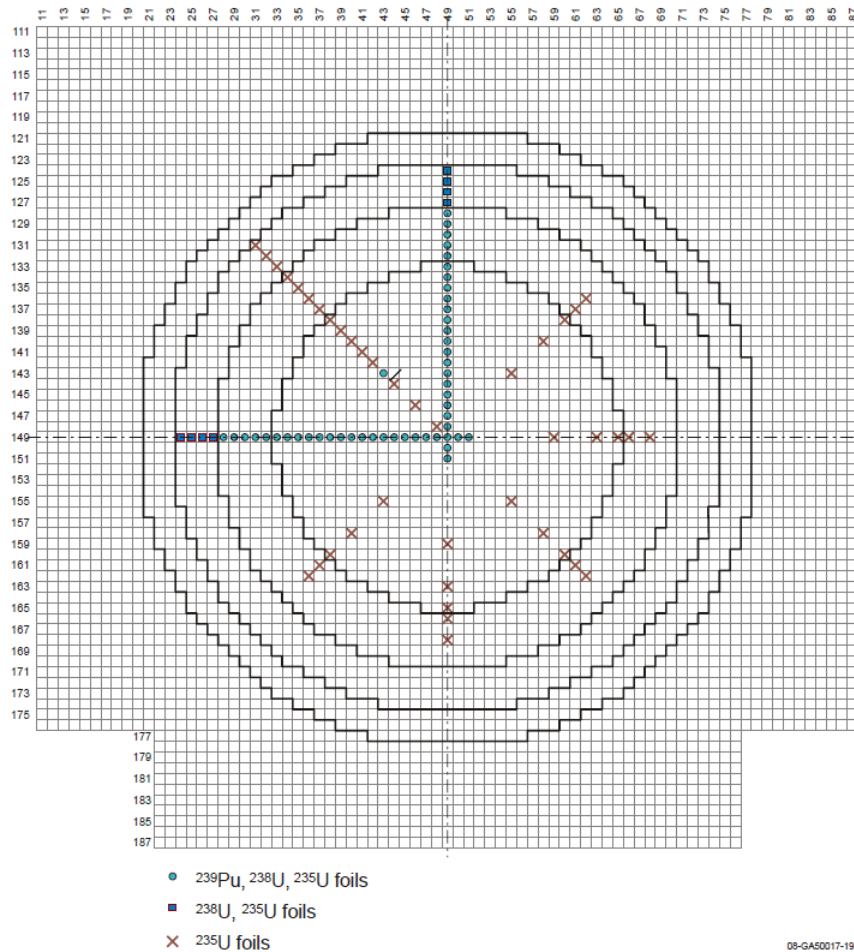


Fig. 1.31 Measured Radial Reaction Rate Positions in ZPPR-9 (Ref.: D. N. Olsen, "ZPPR Assembly 9 – Detailed Work Plan No. 3", ZPR-I-306, Argonne National Laboratory (Jun. 1978).)

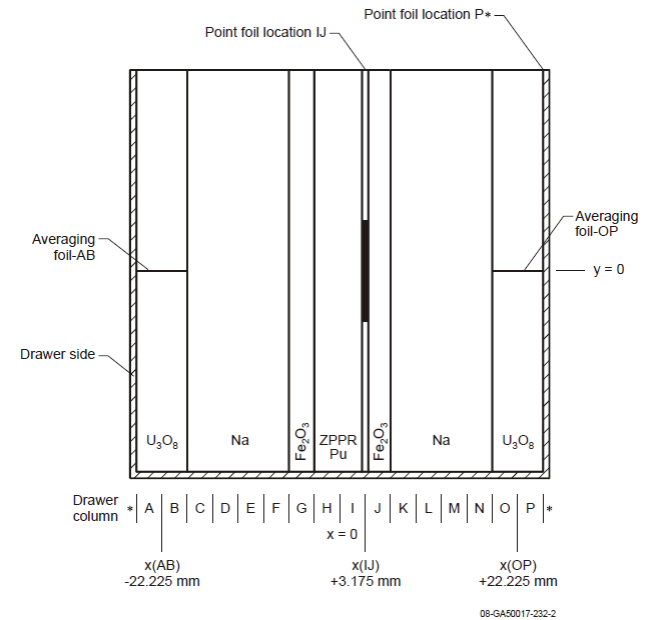
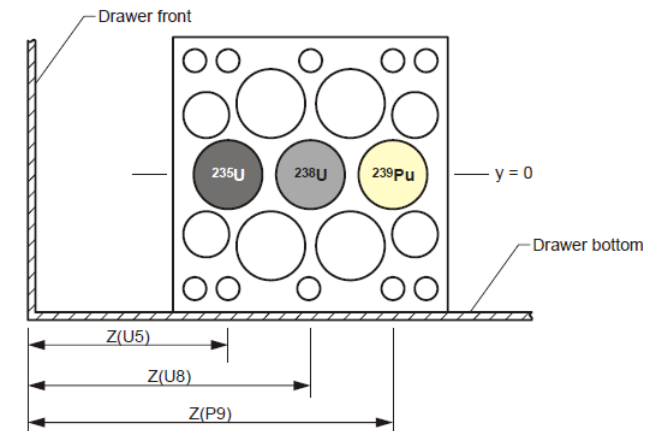
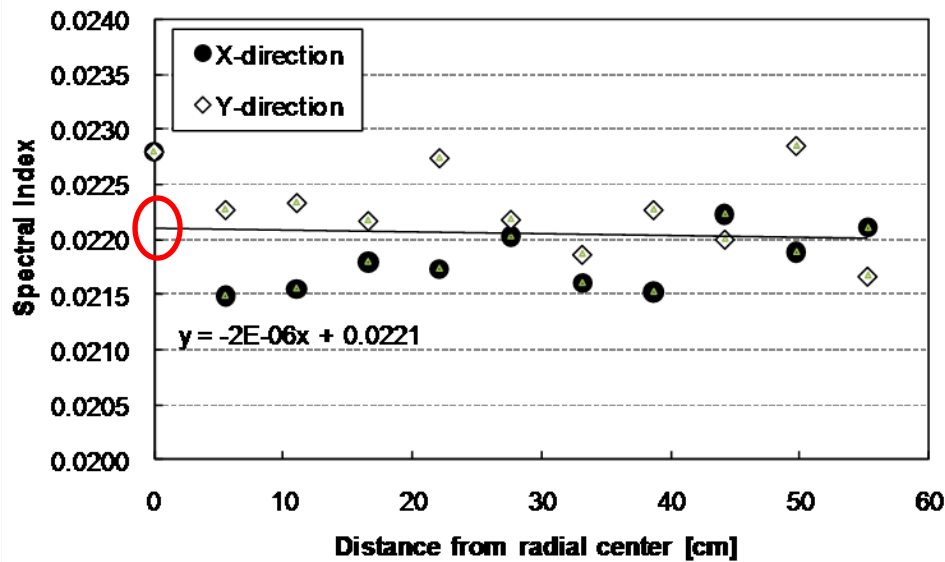


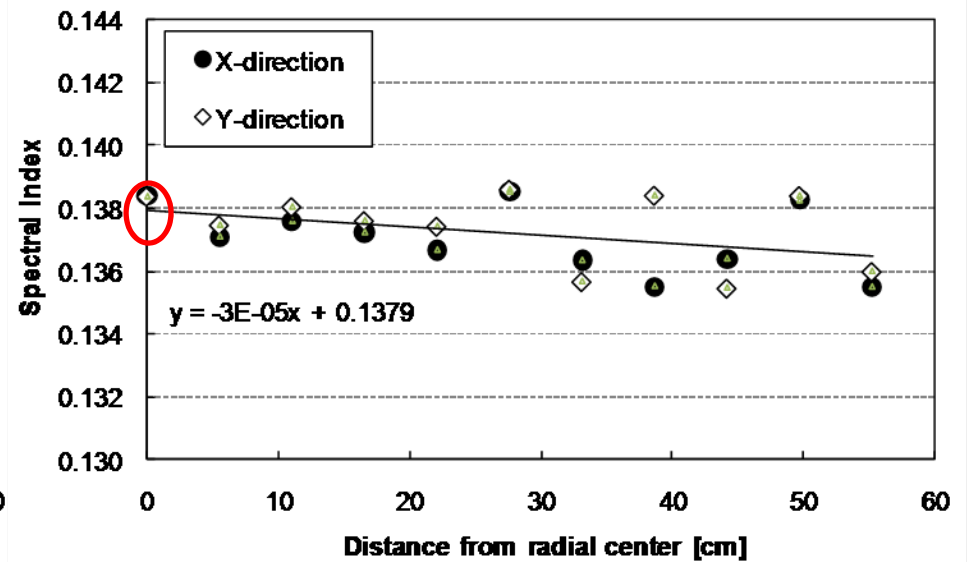
Fig. 1.33 Foil Locations in a Drawer (view from front)



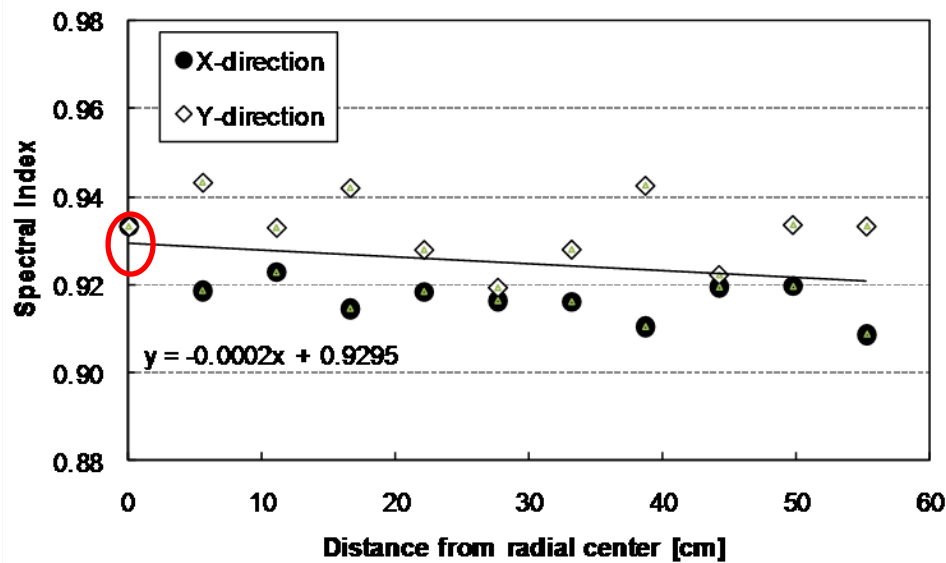
**Fig. Measurement of reaction rate in ZPPR-9**  
 (Radial layout, cross-section of a drawer with irradiation foils)



F28/F25



C28/F25



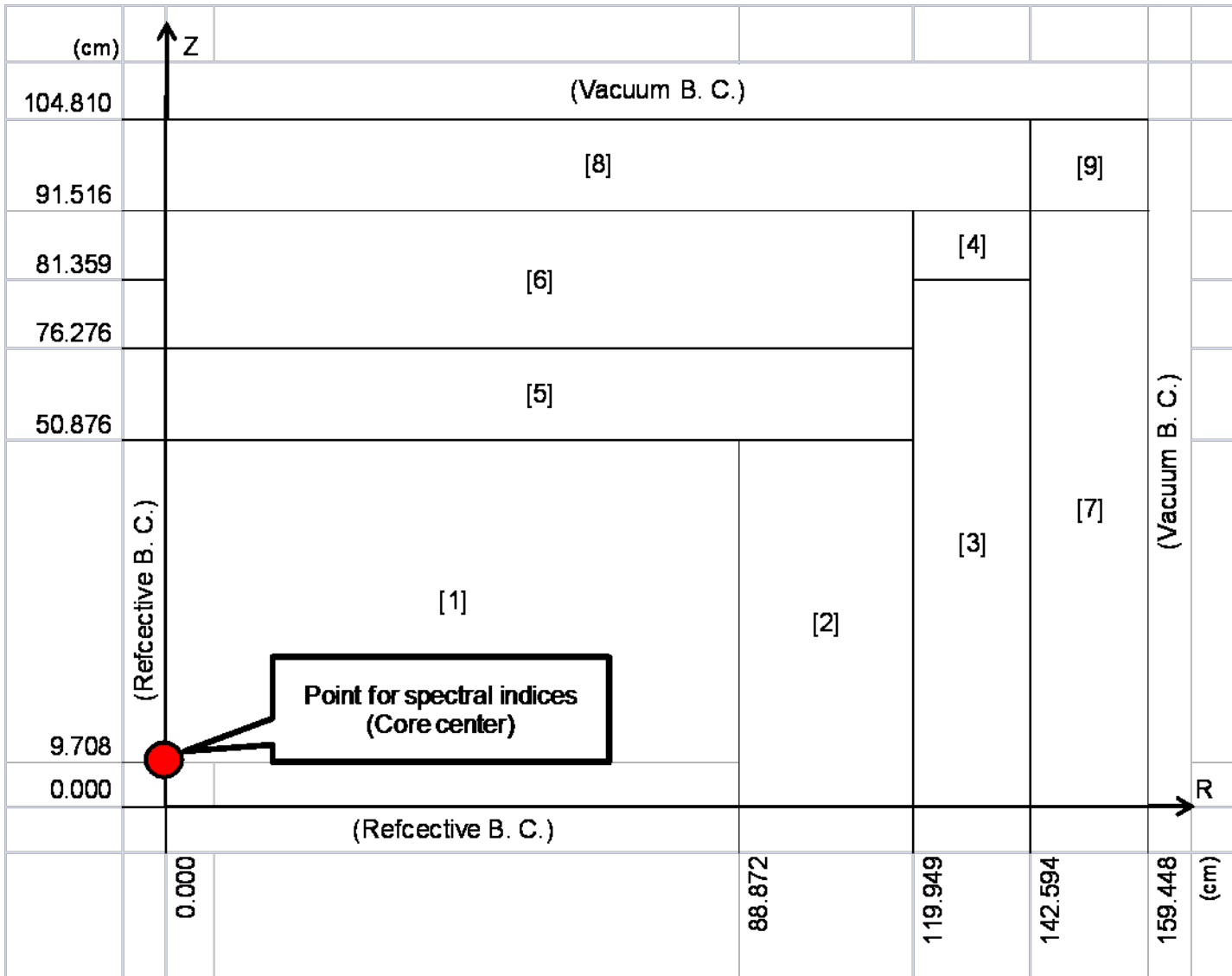
F49/F25

Table Experimental values

Spectral index	New	Previous	
	Core center	Inner core average	Outer core average
F28/F25	0.02210	0.02230	0.02682
F49/F25	0.9295	0.9289	0.9600
C28/F25	0.1379	0.1373	0.1350

**REPLACEMENT**

**Fig. Experimental values of spectral indices on ZPPR-9**



- |     |                        |
|-----|------------------------|
| [1] | Inner Core             |
| [2] | Outer Core             |
| [3] | Radial Blanket (Lower) |
| [4] | Radial Blanket (Upper) |
| [5] | Axial Blanket (Lower)  |
| [6] | Axial Blanket (Upper)  |
| [7] | Radial Reflector       |
| [8] | Axial Reflector        |
| [9] | Matrix                 |

**REPLACEMENT**

**Fig. 2-D RZ-geometry modeling of ZPPR-9  
(For spectral indices)**

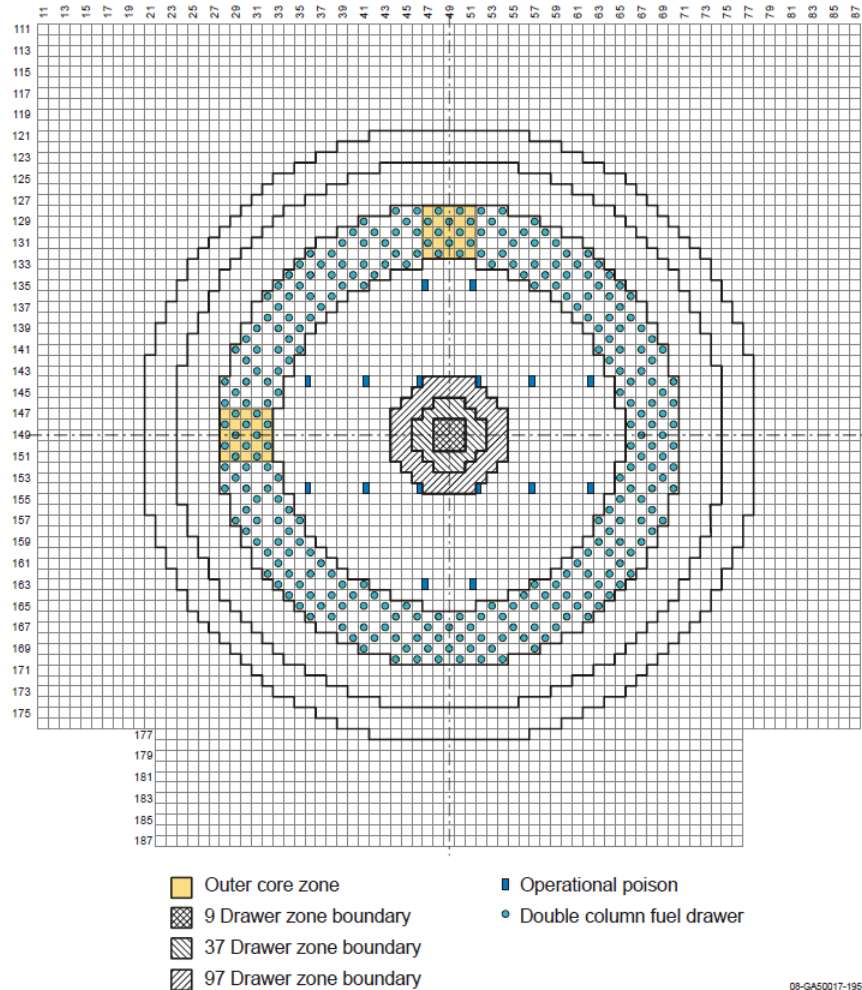


Fig.1.25 ZPPR-9 Sodium Void Zones (Ref.: "ZPPR-9 Monthly Report for October 1978", ZPR-TM-329, Argonne National Laboratory (Oct. 1978).)

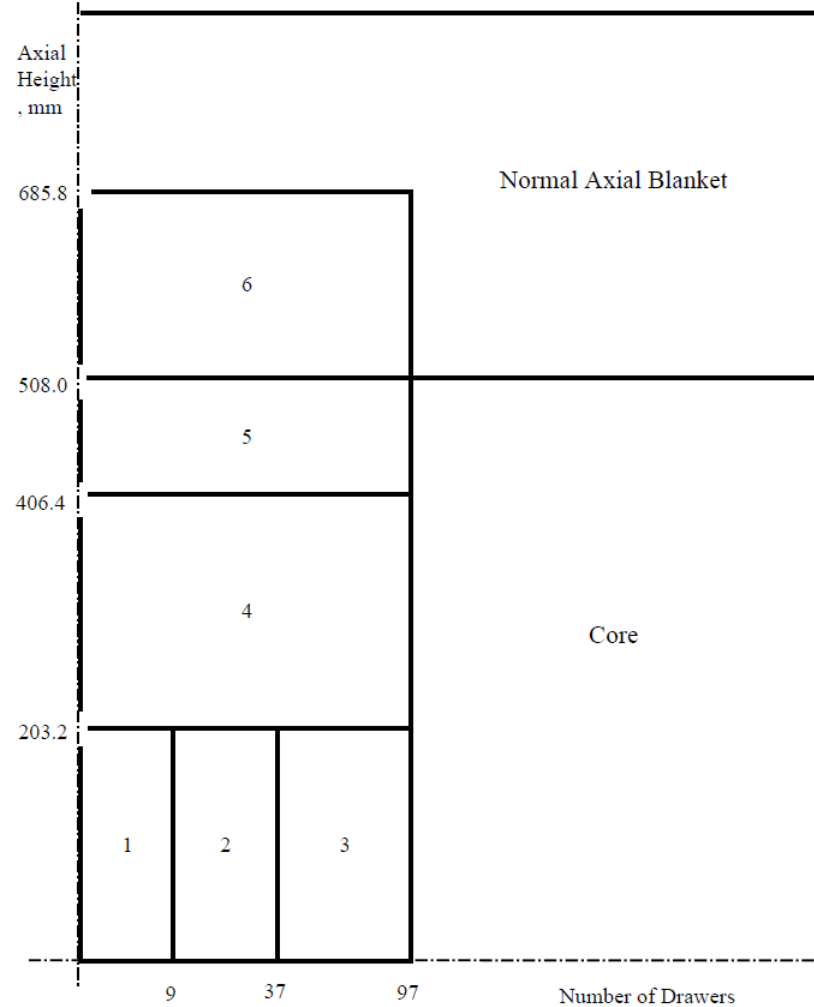
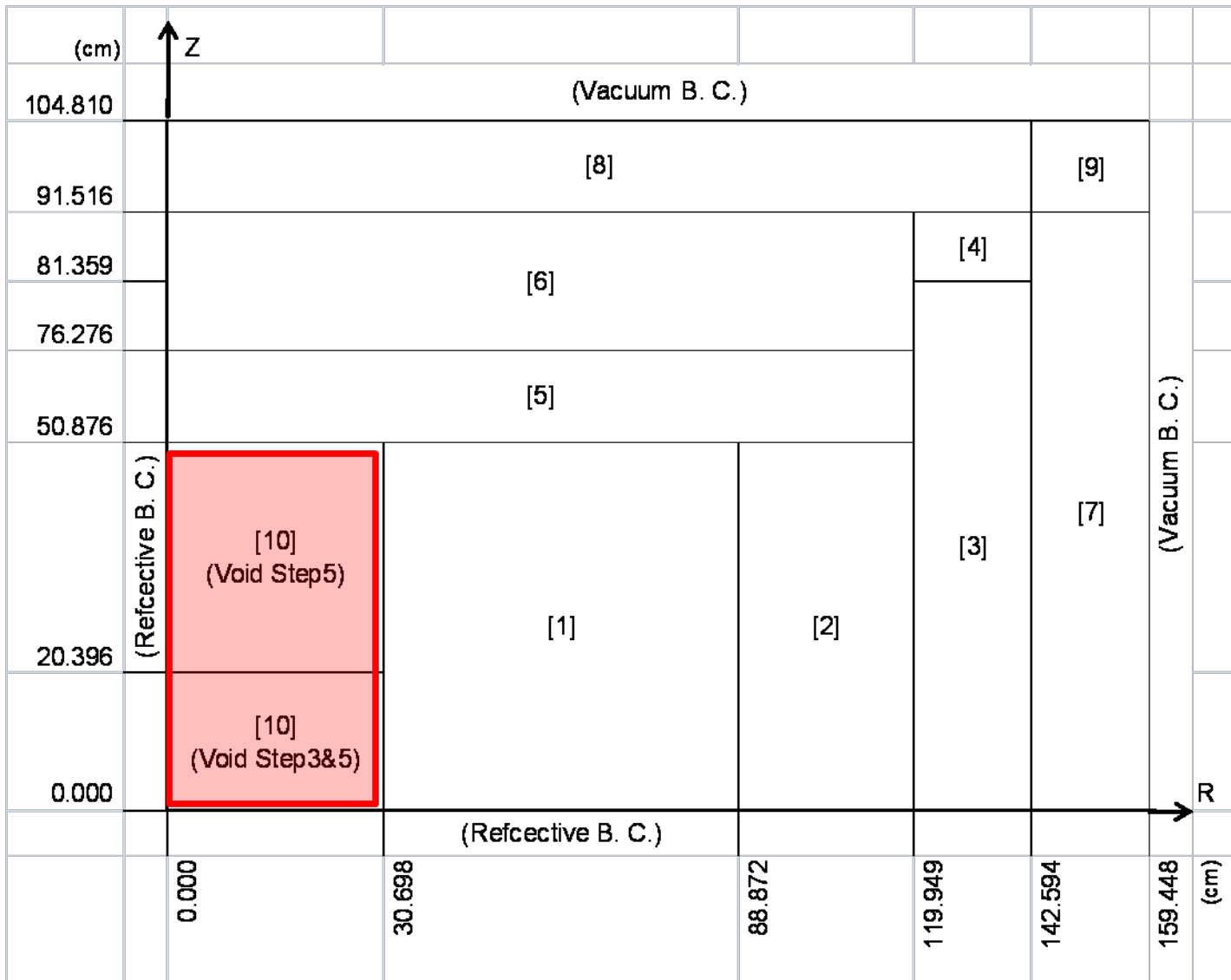


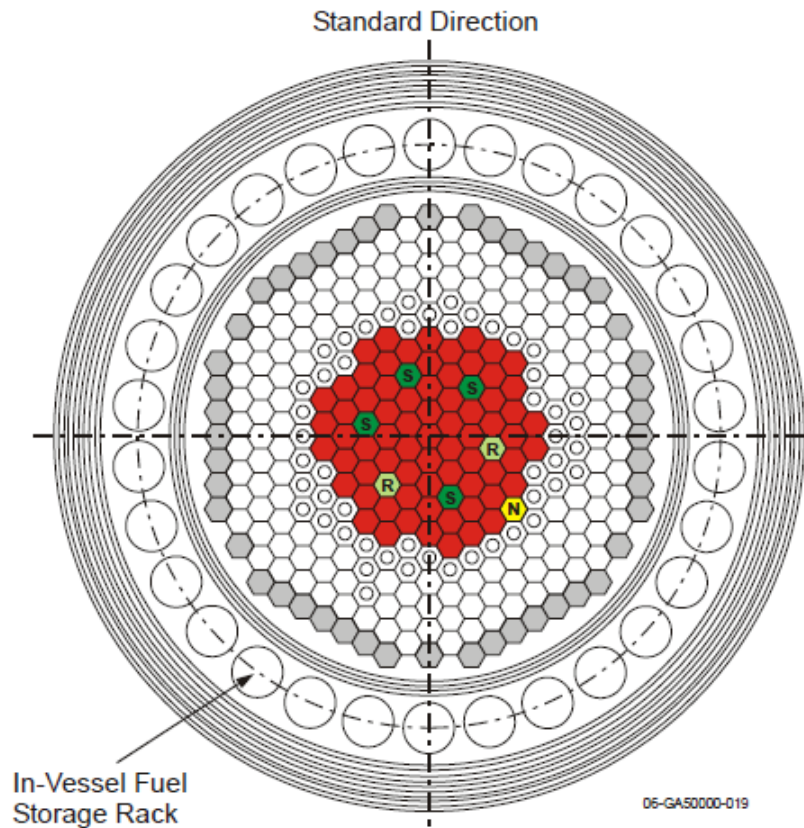
Fig. 1.26 Sequence for Zone Voiding in ZPPR-9 (Ref.: "ZPPR-11 Monthly Report for February 1980", ZPR-TM-361, Argonne National Laboratory (Feb. 1980).)

# Fig. Measurement of sodium void reactivity in ZPPR-9 (Radial layout, axial layout)



- |                            |  |  |
|----------------------------|--|--|
| [1] Inner Core             |  |  |
| [2] Outer Core             |  |  |
| [3] Radial Blanket (Lower) |  |  |
| [4] Radial Blanket (Upper) |  |  |
| [5] Axial Blanket (Lower)  |  |  |
| [6] Axial Blanket (Upper)  |  |  |
| [7] Radial Reflector       |  |  |
| [8] Axial Reflector        |  |  |
| [9] Matrix                 |  |  |
| [10] Na voided Inner Core  |  |  |

**Fig. 2-D RZ-geometry modeling of ZPPR-9  
(For sodium void reactivity)**



- |                                |                       |
|--------------------------------|-----------------------|
| Regulation Rod (2 locations)   | Reflector Subassembly |
| Safety Rod (4 locations)       | Fuel Subassembly      |
| Inner Blanket Fuel Subassembly | Neutron Source        |
| Outer Blanket Fuel Subassembly |                       |

Figure 1.4. Horizontal Cross-section of JOYO MK-I Core.

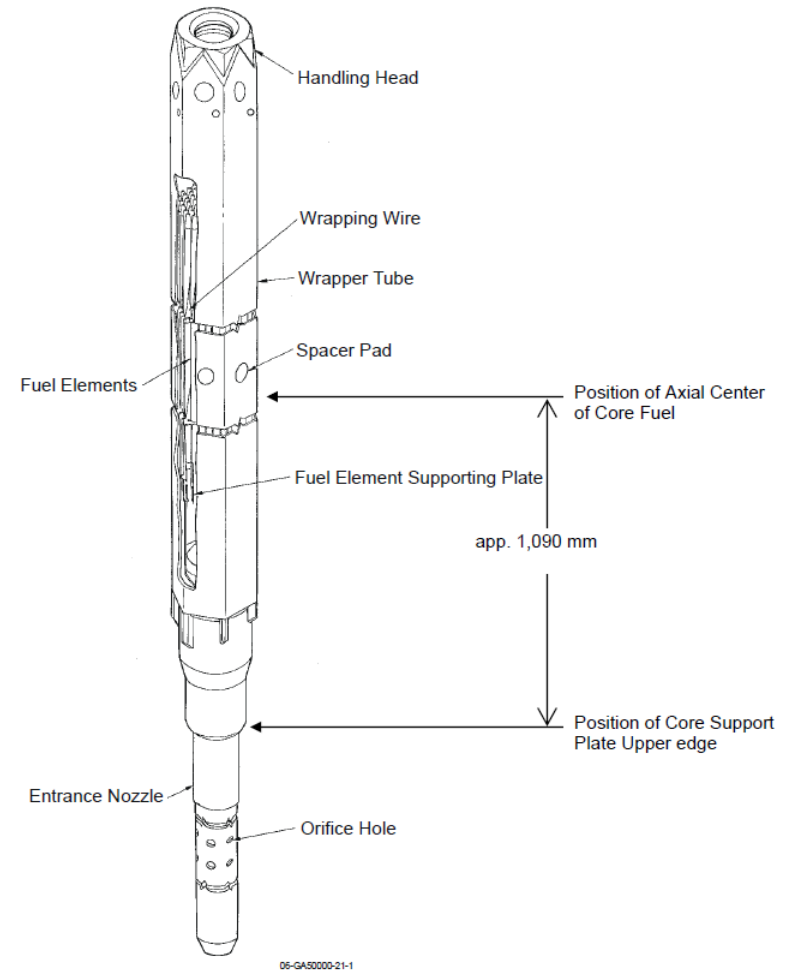
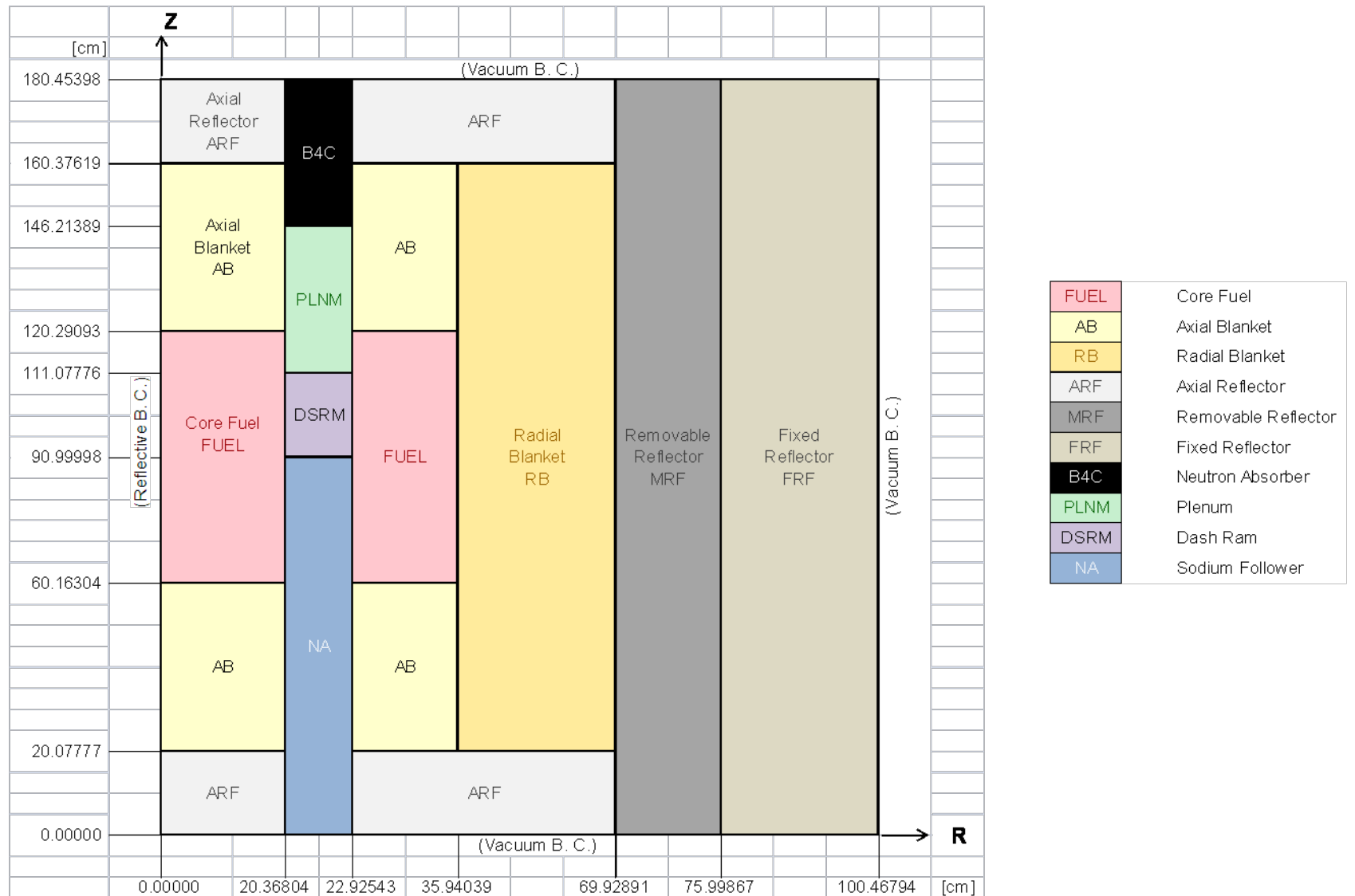


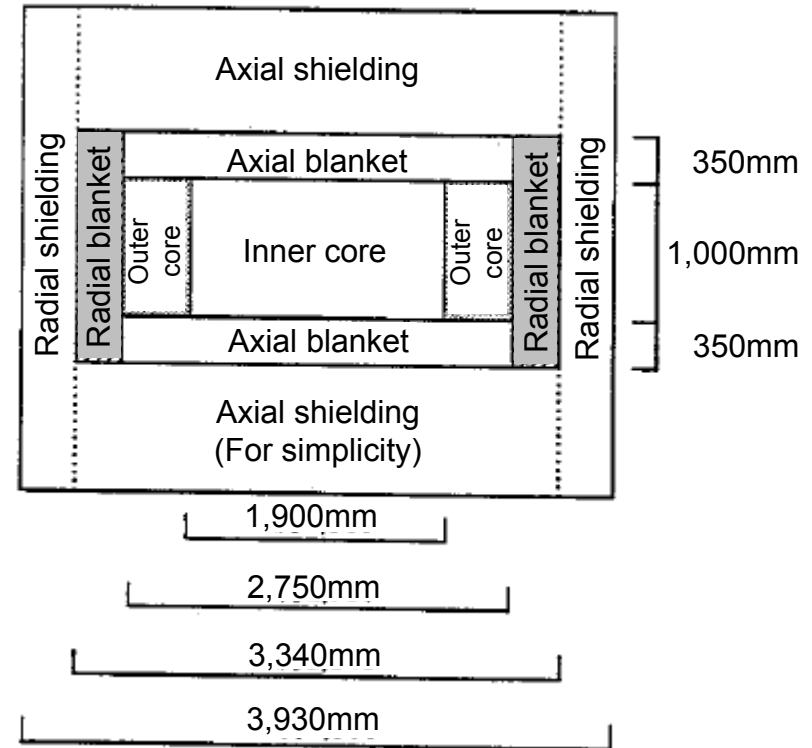
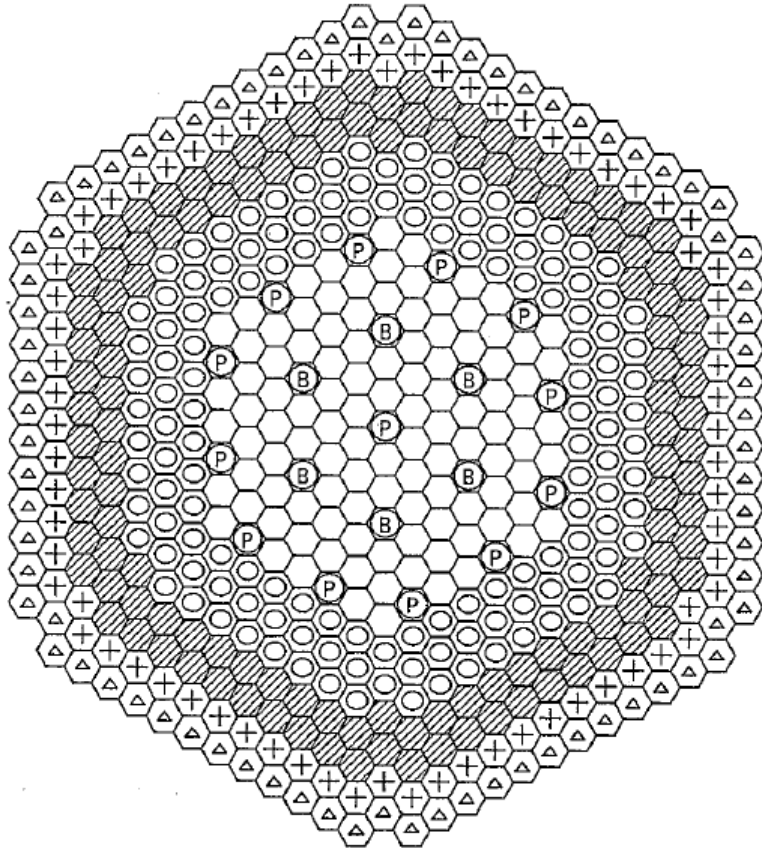
Figure 1.6. Schematic View of Core Fuel Subassembly.

## Fig. Configuration of Joyo MK-I (Radial layout, schematic view of fuel subassembly)



**Fig. 2-D RZ-geometry modeling of Joyo MK-I  
(For critical mass)**

# Brief Description of JAEA FBR (600MWe FBR Core)

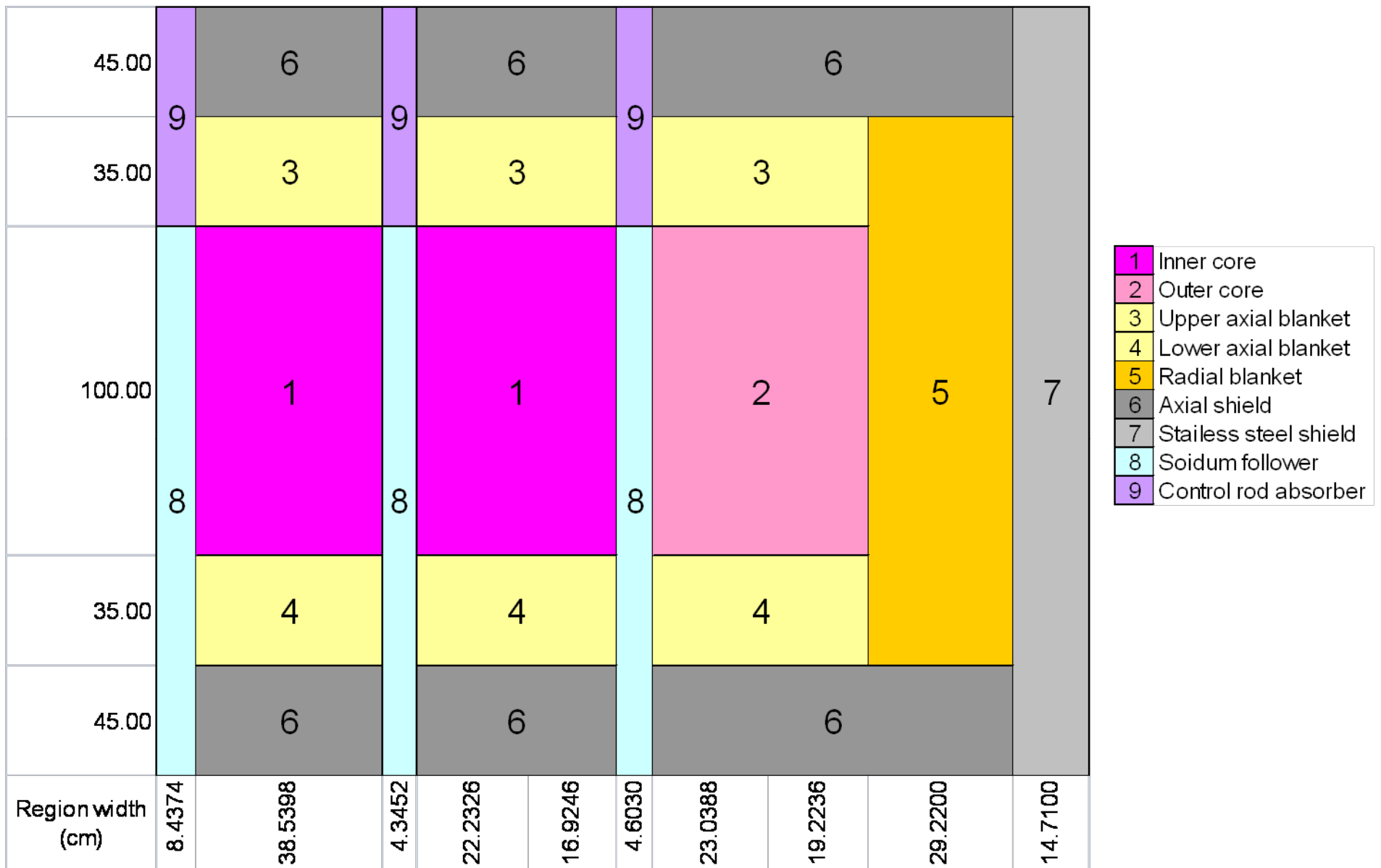


- Inner core (Pu\* fraction of 17wt%)
- ⊗ Outer core (Pu\* fraction of 21wt%)
- ▨ Radial blanket
- ⊕ Stainless steel shielding
- △ B<sub>4</sub>C shielding
- Ⓟ Primary control rod
- Ⓡ Back-up control rod

\*Pu isotopic composition: 3/53/25/12/7wt%  
(238/239/240/241/242)

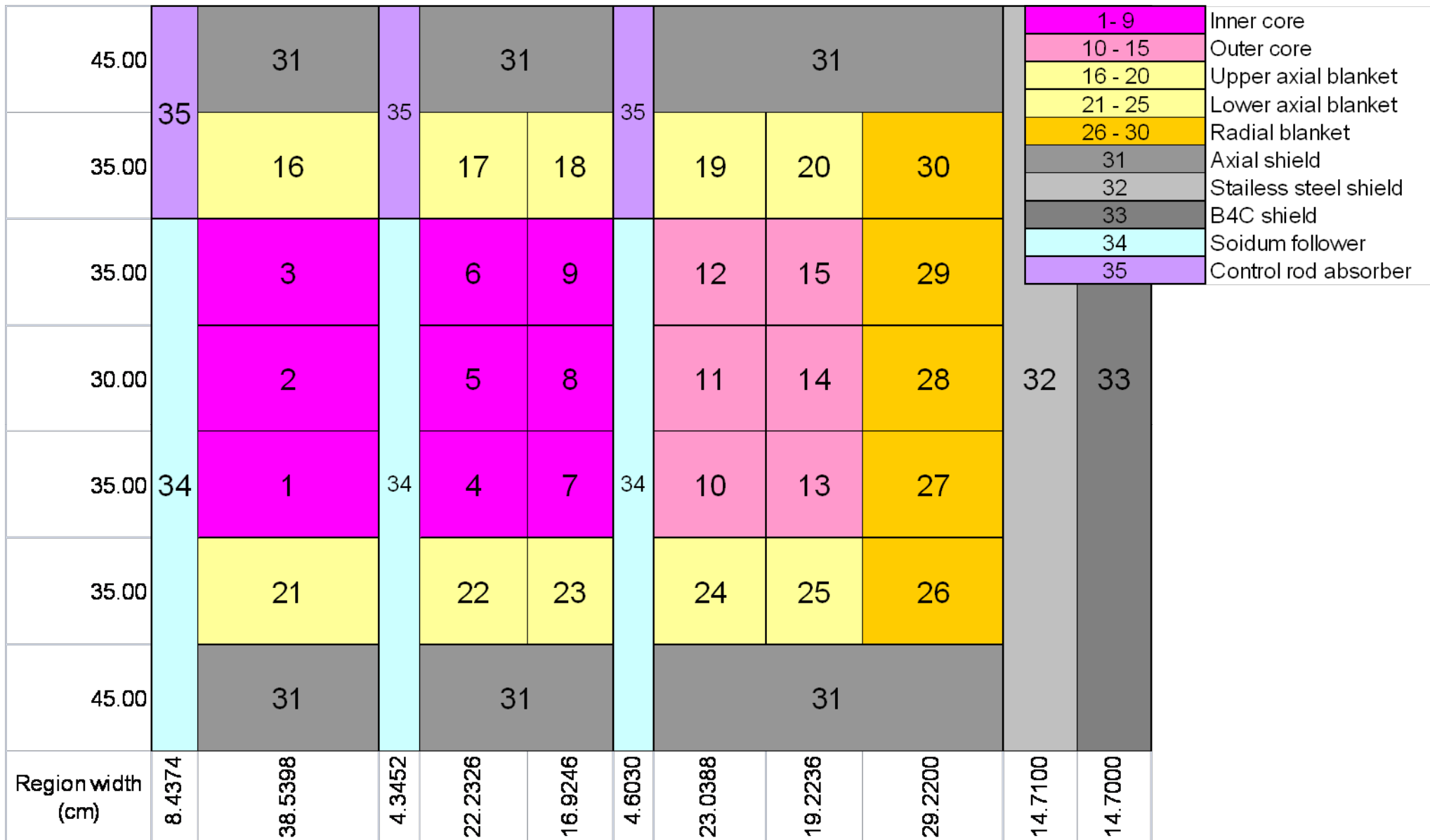
## Major specification

Thermal / Electric power: 1,600MWt / 600MWe  
 Fuel type: Plutonium-uranium mixed oxide  
 Operation cycle length: 375 days  
 Refueling batch (core): 3  
 Discharge burnup (core): 86GWd/t  
 Burnup reactivity swing: 2.7%Δk/kk'  
 Breeding ratio: 1.2



- 1 Inner core
- 2 Outer core
- 3 Upper axial blanket
- 4 Lower axial blanket
- 5 Radial blanket
- 6 Axial shield
- 7 Stainless steel shield
- 8 Sodium follower
- 9 Control rod absorber

**Fig. 2-D RZ-geometry modeling of JAEA FBR**  
(Reference model)



**Fig. 2-D RZ-geometry modeling of JAEA FBR**  
(Detailed model)