



COMMARA-3

Processed ENDF/B-VII.1 covariance library

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COMMARA-3.0 - ENDF/B-VII.1 Covariance Materials

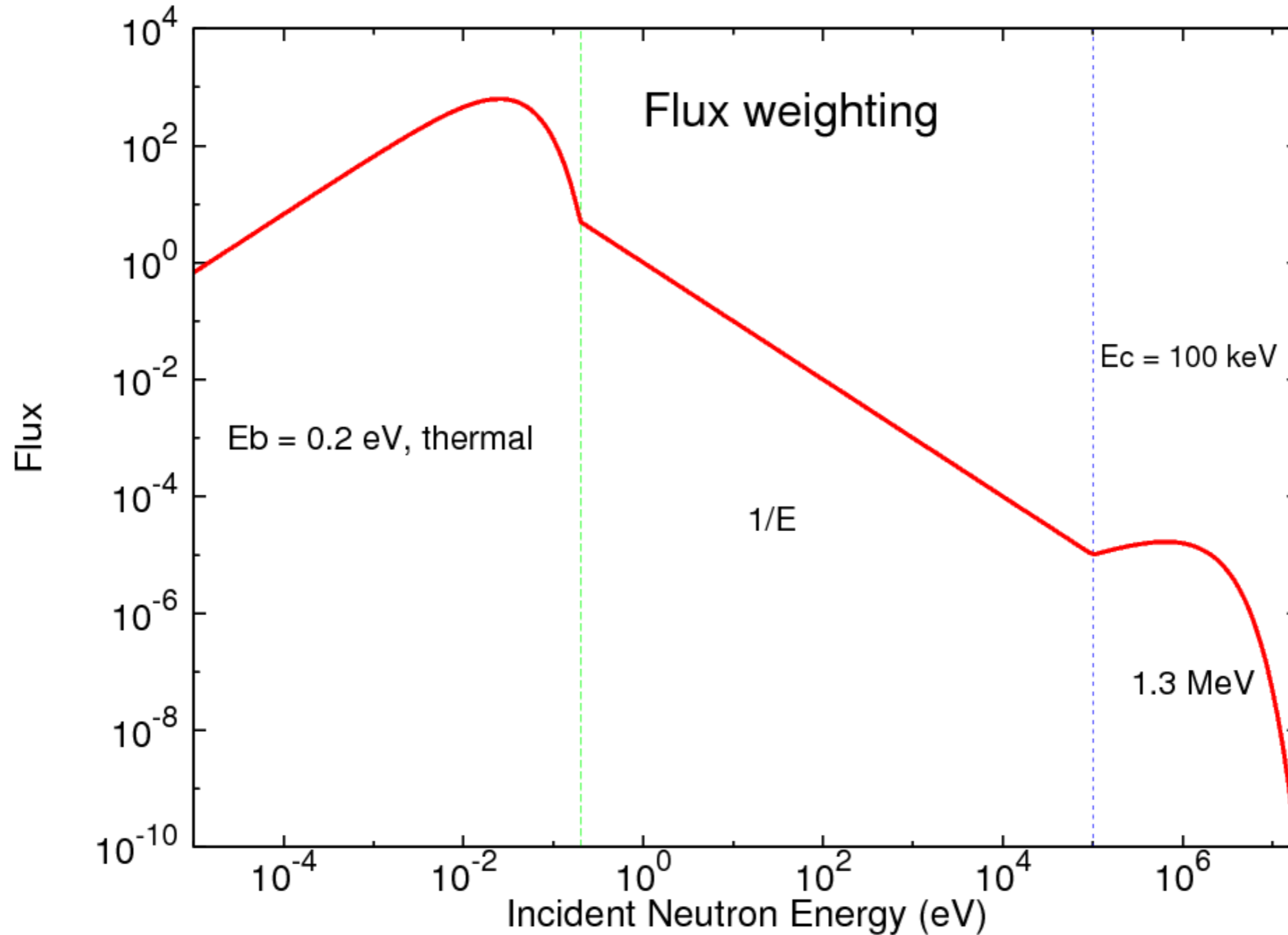
184 materials: 12 Light, 99 structural, 73 Actinides

- ^1H , ^2H , ^4He , ^6Li , ^7Li , ^9Be , ^{10}B , ^{11}B , ^{12}C , ^{15}N , ^{16}O , ^{19}F , ^{24}Mg , ^{25}Mg , ^{26}Mg , ^{27}Al , ^{28}Si , ^{29}Si , ^{30}Si , ^{35}Cl , ^{37}Cl , ^{39}K , ^{41}K , ^{46}Ti , ^{47}Ti , ^{48}Ti , ^{49}Ti , ^{50}Ti , ^{50}Cr , ^{52}Cr , ^{53}Cr , ^{54}Cr , ^{55}Mn , ^{54}Fe , ^{56}Fe , ^{57}Fe , ^{59}Co , ^{58}Ni , ^{60}Ni , ^{89}Y , ^{90}Zr , ^{91}Zr , ^{92}Zr , ^{93}Zr , ^{94}Zr , ^{95}Zr , ^{96}Zr , ^{95}Nb , ^{92}Mo , ^{94}Mo , ^{95}Mo , ^{96}Mo , ^{97}Mo , ^{98}Mo , ^{100}Mo , ^{99}Tc , ^{101}Ru , ^{102}Ru , ^{103}Ru , ^{104}Ru , ^{106}Ru , ^{105}Pd , ^{107}Pd , ^{108}Pd , ^{109}Ag , ^{127}I , ^{129}I , ^{131}Xe , ^{132}Xe , ^{134}Xe , ^{133}Cs , ^{135}Cs , ^{139}La , ^{141}Ce , ^{141}Pr , ^{143}Nd , ^{145}Nd , ^{146}Nd , ^{148}Nd , ^{147}Pm , ^{149}Sm , ^{151}Sm , ^{142}Sm , ^{153}Eu , ^{155}Eu , ^{152}Gd , ^{153}Gd , ^{154}Gd , ^{155}Gd , ^{156}Gd , ^{157}Gd , ^{158}Gd , ^{160}Gd
- ^{166}Er , ^{167}Er , ^{168}Er , ^{170}Er , ^{180}W , ^{182}W , ^{183}W , ^{184}W , ^{186}W , ^{191}Ir , ^{193}Ir , ^{197}Au , ^{204}Pb , ^{206}Pb , ^{207}Pb , ^{208}Pb , ^{209}Bi , ^{225}Ac , ^{226}Ac , ^{227}Ac , ^{227}Th , ^{229}Th , ^{230}Th , ^{231}Th , ^{232}Th , ^{233}Th , ^{234}Th , ^{229}Pa , ^{230}Pa , ^{232}Pa , ^{230}U , ^{231}U , ^{232}U , ^{233}U , ^{234}U , ^{235}U , ^{236}U , ^{238}U , ^{234}Np , ^{235}Np , ^{236}Np , ^{237}Np , ^{238}Np , ^{239}Np , ^{236}Pu , ^{237}Pu , ^{238}Pu , ^{239}Pu , ^{240}Pu , ^{241}Pu , ^{242}Pu , ^{244}Pu , ^{246}Pu , ^{240}Am , ^{241}Am , $^{242\text{m}1}\text{Am}$, ^{243}Am , ^{240}Cm , ^{241}Cm , ^{242}Cm , ^{243}Cm , ^{244}Cm , ^{245}Cm , ^{246}Cm , ^{248}Cm , ^{249}Cm , ^{250}Cm , ^{245}Bk , ^{246}Bk , ^{247}Bk , ^{248}Bk , ^{250}Bk , ^{246}Cf , ^{249}Cf , ^{250}Cf , ^{251}Cf , ^{252}Cf , ^{253}Cf , ^{254}Cf , ^{251}Es , ^{252}Es , ^{253}Es , ^{254}Es , $^{254\text{m}1}\text{Es}$, ^{255}Es , ^{255}Fm

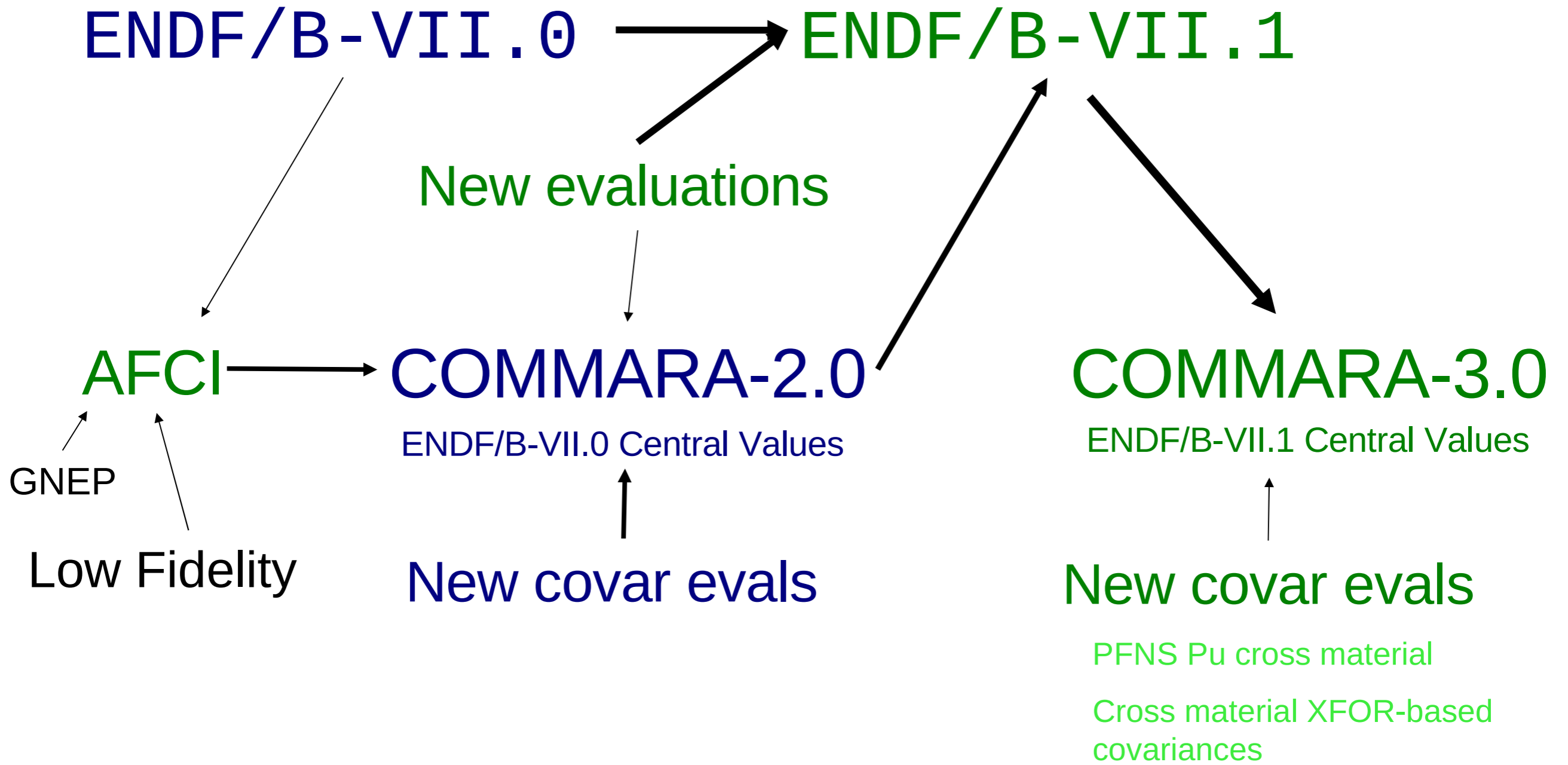
Scope

- Associated with neutron cross sections from **ENDF/B-VII.1**
- Multigroup (processed) covariances with uniform lethargy energy groups (33 groups, flux: thermal-1/E-fiss spec)
- Reaction channels:
 - (n,el), (n,inl), (n,2n), (n, γ)
 - (n,f) & nubar MF31 (73)
 - prompt fission neutron spectra (PFNS, $^{238,239,240}\text{Pu}$ LANL, JENDL-4 MF35 (85), plus cross-material covariances (LANL))
 - Mubars MF34 (116 & additional from JENDL)

NJOY flux weighting



Recent Covariance Evolution



Quality Assurance

- Web-based Sigma-QA (A. Sonzogni) allows visual and also quantitative inspection of:
 - Differential uncertainties (dynamic)
 - Integral uncertainties (static)
- UnCor applied to full library, performs 8 tests, warnings for possible problems including:
 - small uncertainties: $(n,tot) < 1\%$, (n,el) and $(n,\gamma) < 2\%$, etc.
 - non-positive-definite matrices (fixable for all but PFNS)
 - PFNS covariance not summing to zero (not usually a problem)
- non-positive-definite matrices are usually fixable by slightly reducing the off-diagonal elements. If not, slightly more drastic measures may be required.

COMMARA Checklist

- Test for small uncertainties:

Uncertainty test 1

Total with small
uncertainties : 174

Total with too large
uncertainties : 91

Reaction	Max % Unc
(n,total)	1.0
(n,elas)	2.0
(n,inel)	3.0
(n,2n)	3.0
(n,f)	0.7
(n,g)	2.0
Total nubar	0.7
Prompt nubar	0.7
other	2.0

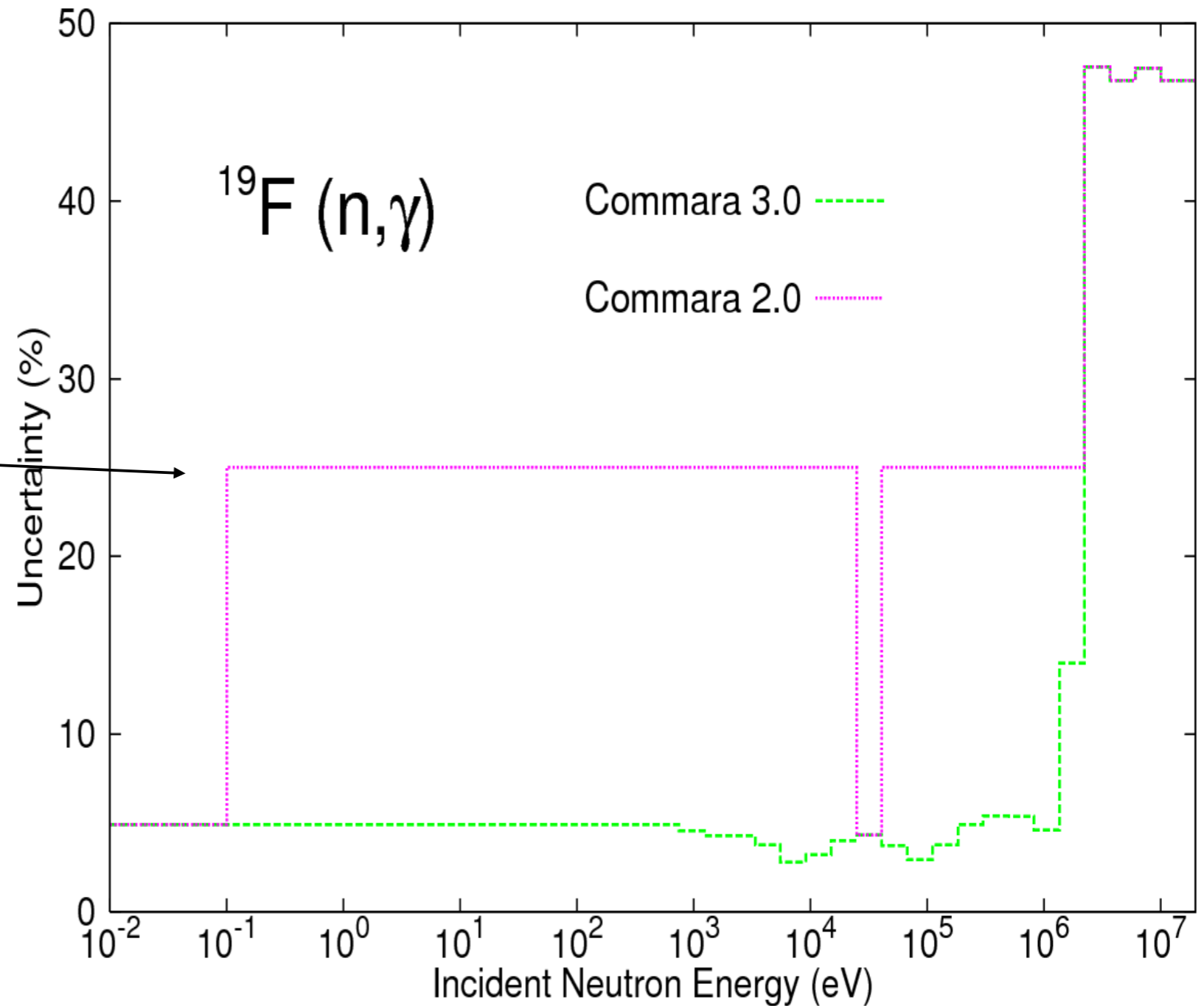
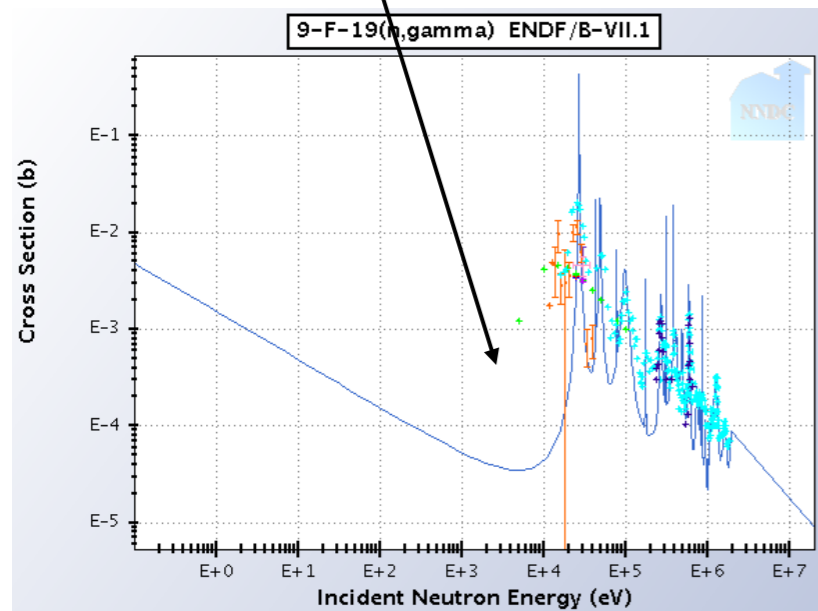
COMMARA Checklist

- 2nd test for too small uncertainties:
If cross section $< 3\text{mb}$ \Rightarrow uncert $\geq 25\%$
Total materials : 128
- Optical model peaks in elastic: 44
- Zero unc with non-zero cross section: 90
- Peaks/jumps in uncertainties: 33 (more than factor of 8)
- Negative eigenvalues : only PFNS (all $\sim 2\text{-}3\text{E-}3$, roundoff err)

^{19}F

3.0: LANL MF33 estimates

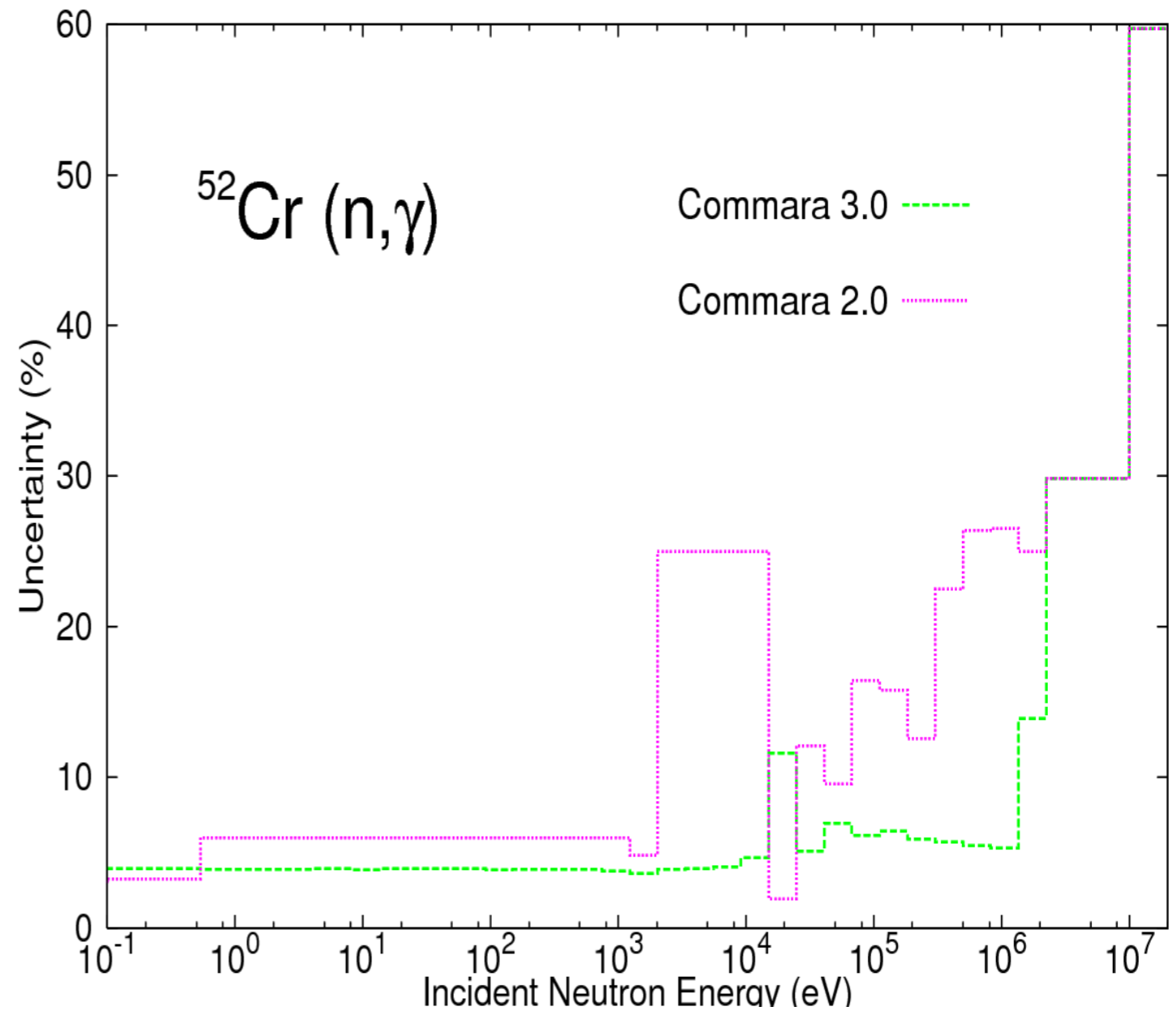
2.0: LANL estimates
with 25% unc limit when
 $\sigma < 3\text{mb}$



^{52}Cr

3.0: New ORNL RRR evaluation

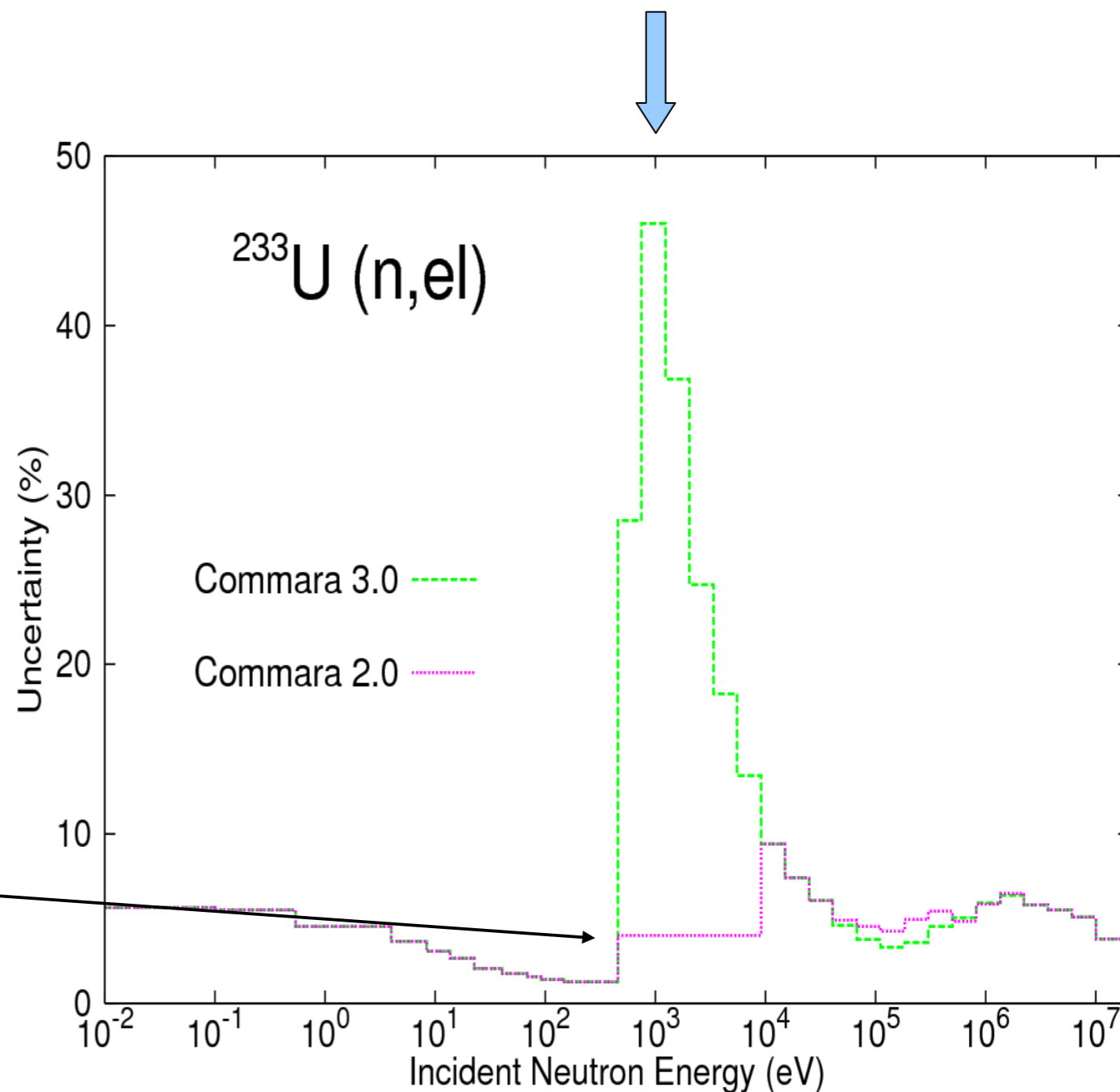
2.0: Estimate using kernel approximation



^{233}U

3.0: Updated LANL evaluation

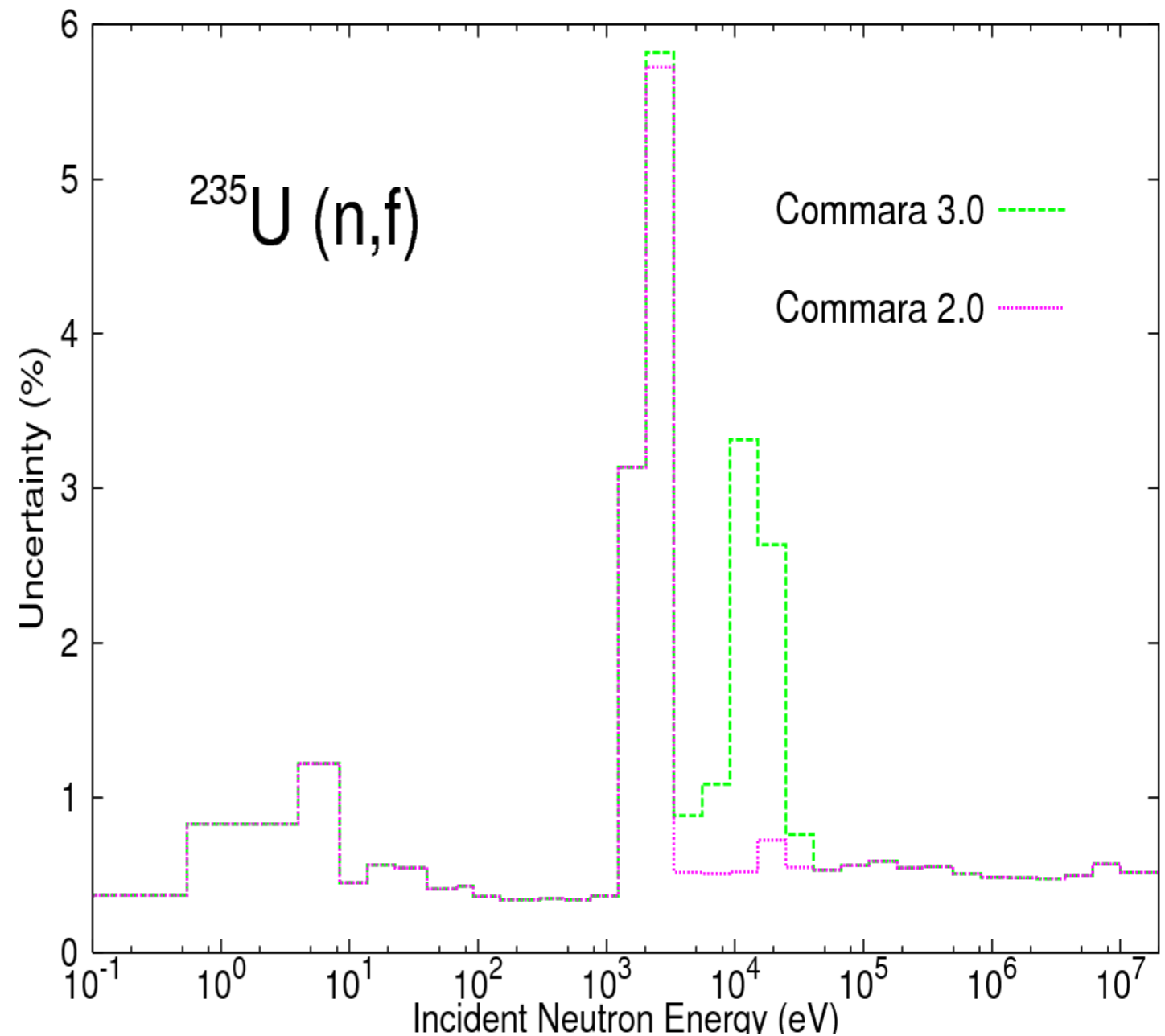
2.0: LANL evaluation, optical peak truncated to 4%



^{235}U

3.0: Updated MF33
V.G. Pronyaev,
IPPE/LANL

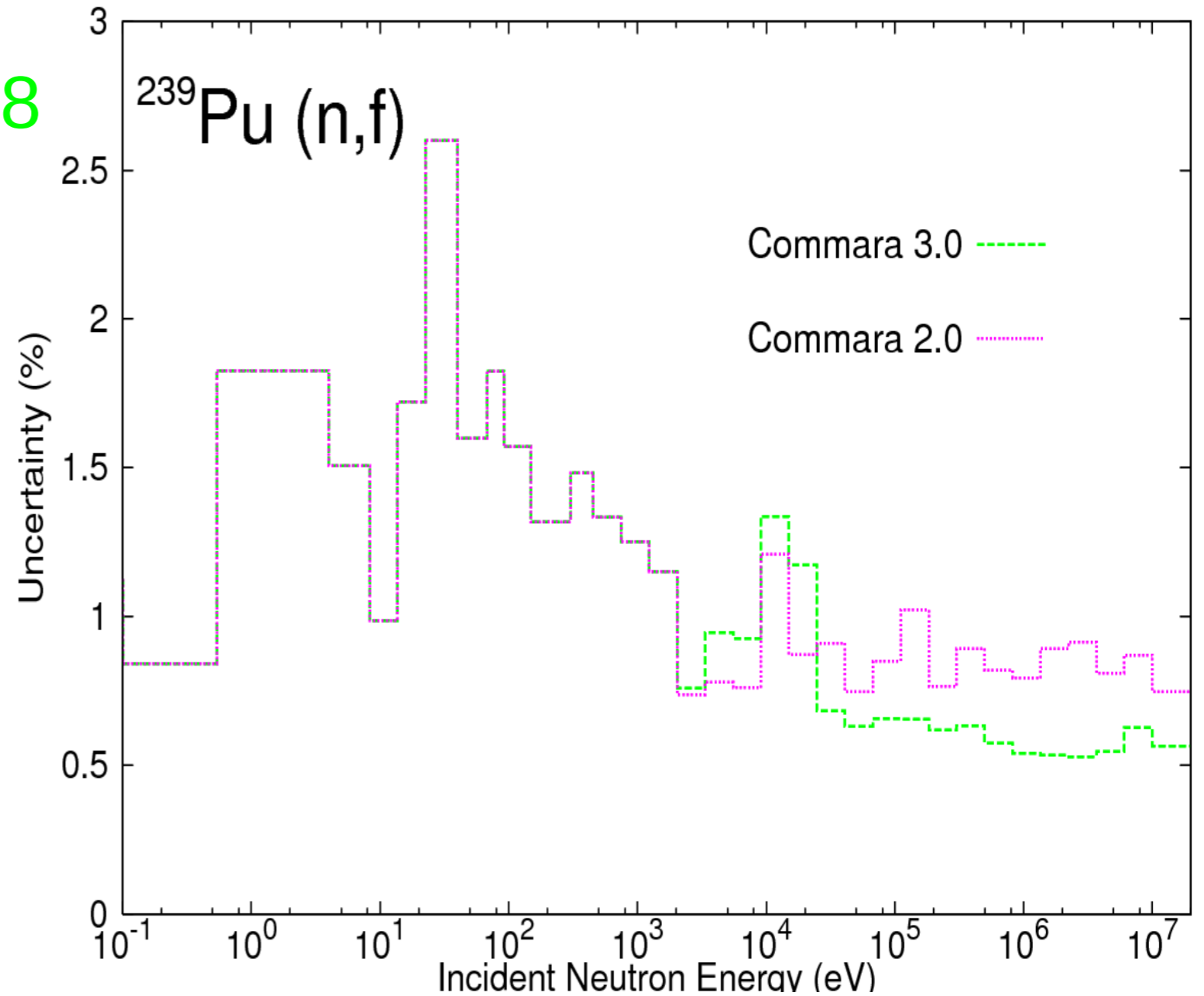
2.0: MF33 covariances
LANL/ORNL



^{239}Pu

3.0: Updated MF33/MT18 standards covariances, V.G. Pronyaev IPPE

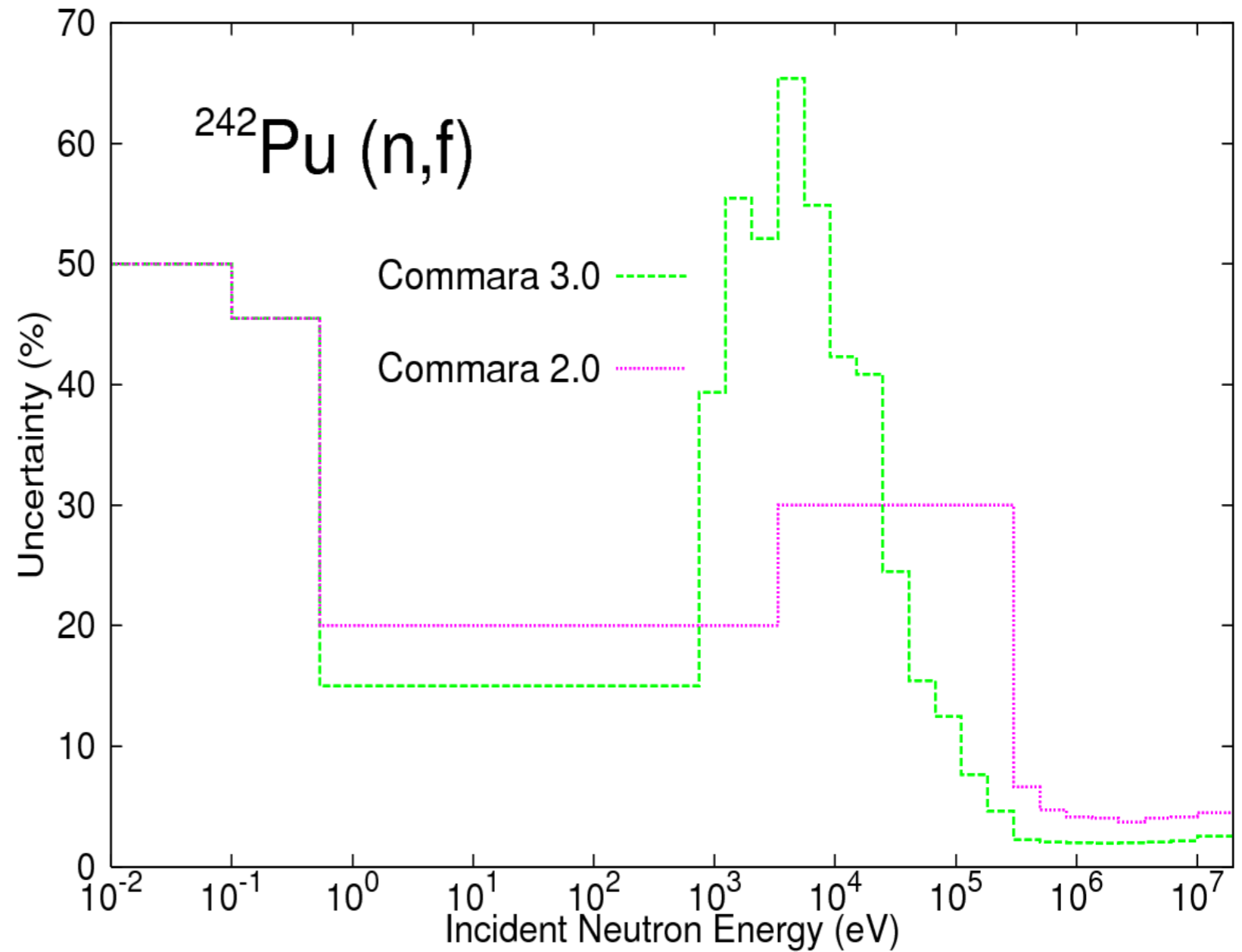
2.0: MF33 evaluation by LANL/ORNL



^{242}Pu

3.0: adopted
JENDL-4

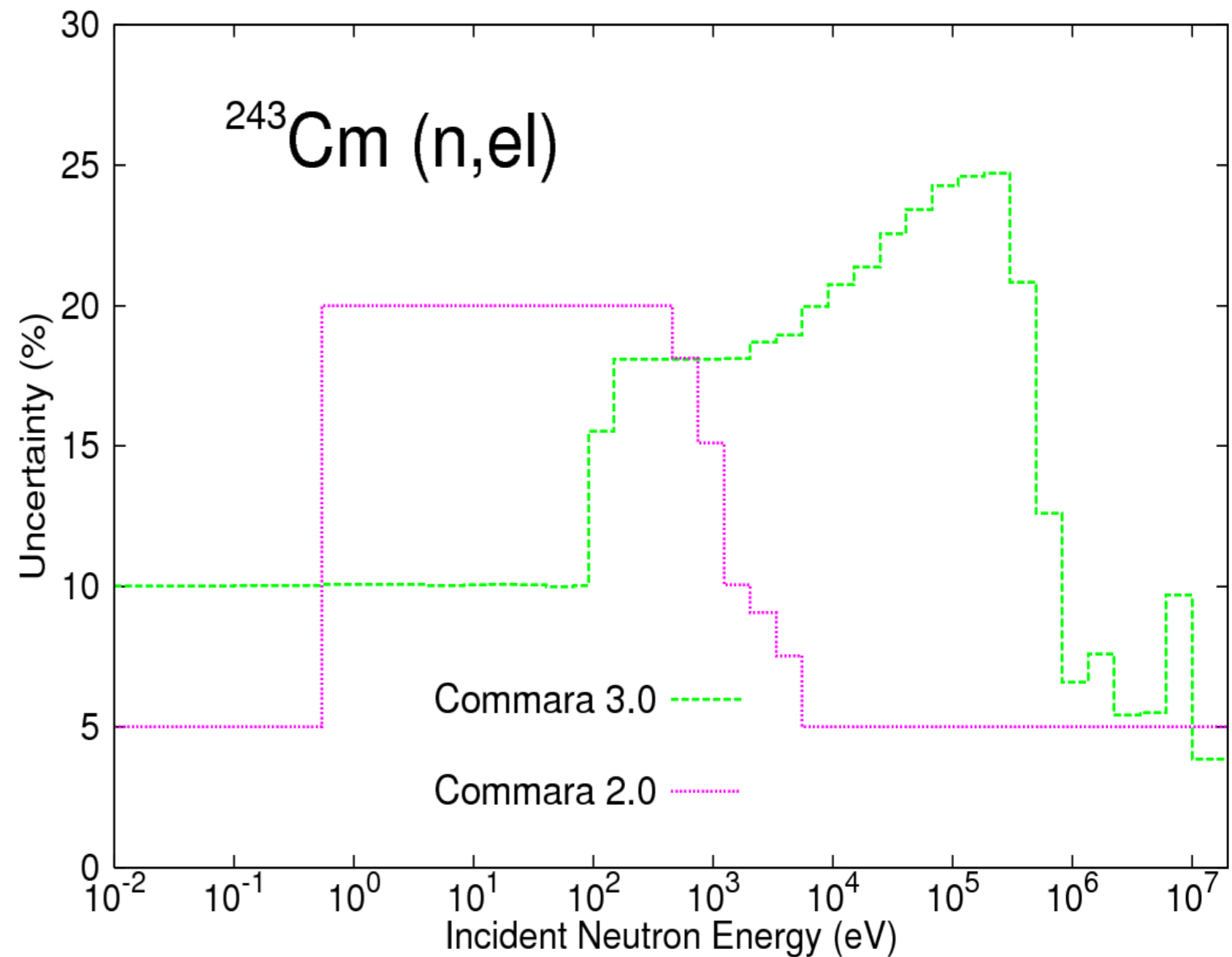
2.0: Empire/Kalman in
fast region.
Low-fidelity estimates in
thermal/RRR.



^{243}Cm

3.0: adopted
JENDL-4

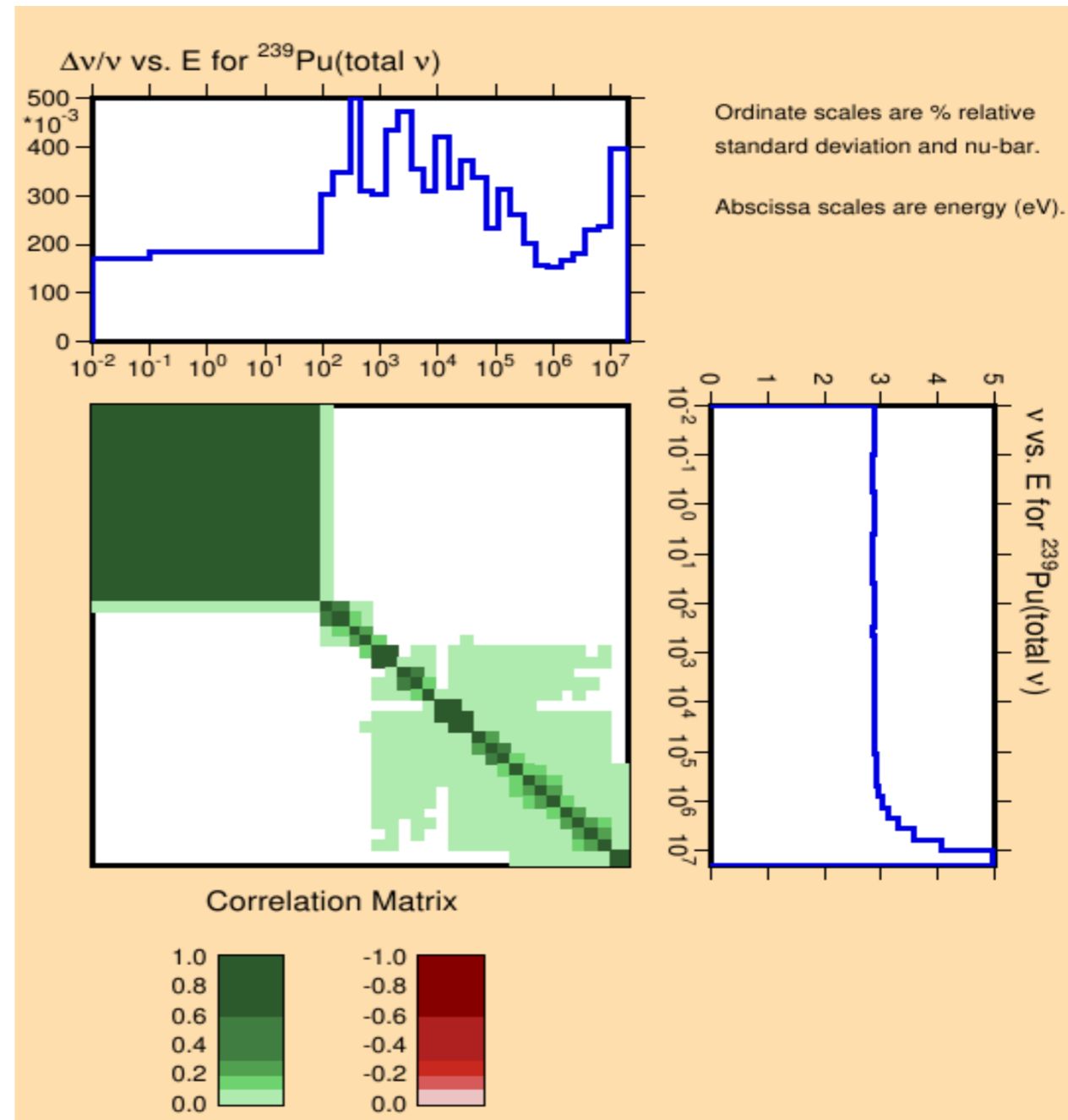
2.0: default
Empire/Kalman with
Low-fidelity estimates in
thermal/RRR



Nubar covariances

^{239}Pu - LANL

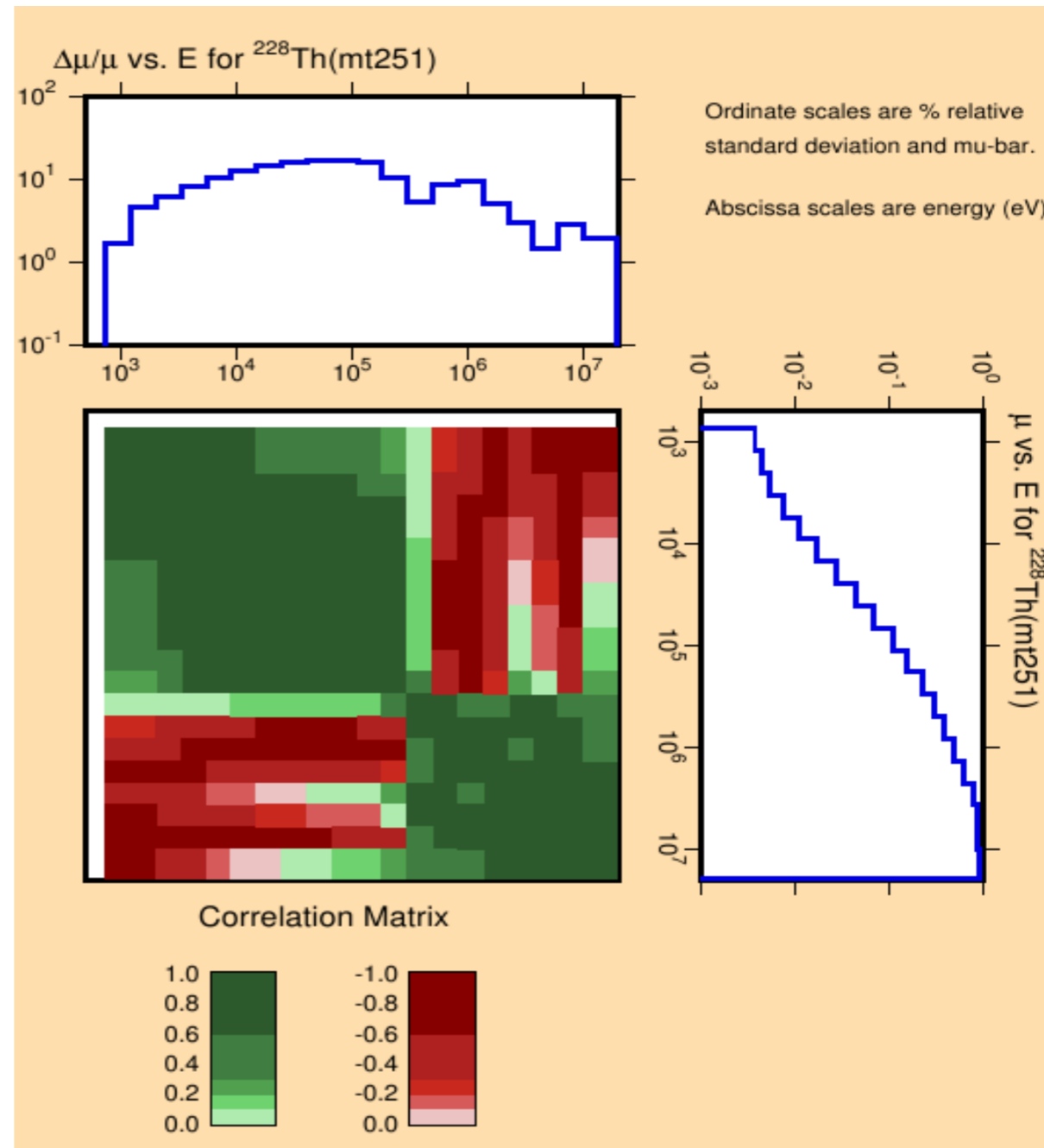
Total materials
with MF31: 73



Mubar covariances

^{228}Th
JENDL-4

Most mubar
covariances in
ENDF/B-VII.1 from
JENDL



Missing Mubars (MF34)

- ^{233}U , ^{234}U , ^{235}U , ^{236}U , ^{238}U
- ^{237}Np
- ^{238}Pu , ^{239}Pu , ^{240}Pu , ^{241}Pu
- ^{241}Am , $^{242\text{m}1}\text{Am}$, ^{243}Am
- $^{242-246}\text{Cm}$

Possible Sources:
LANL
Empire estimates
JENDL

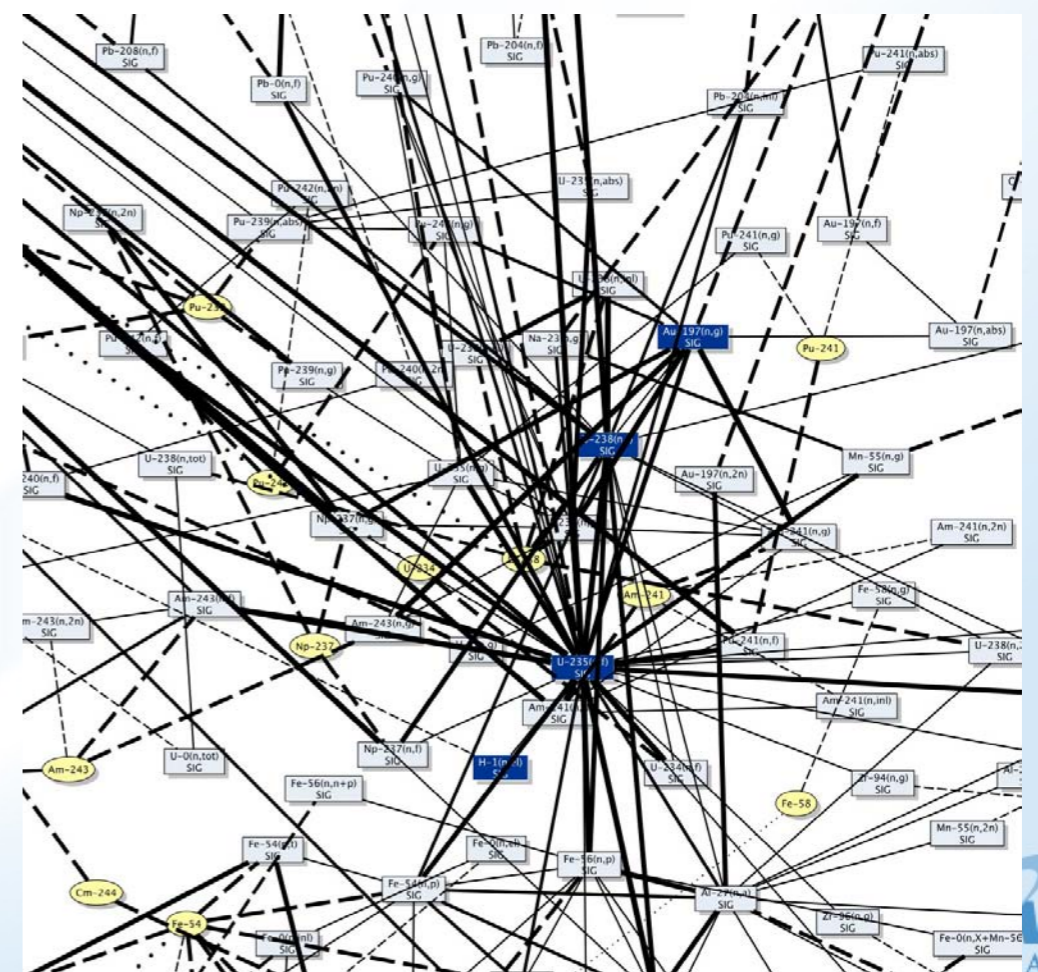
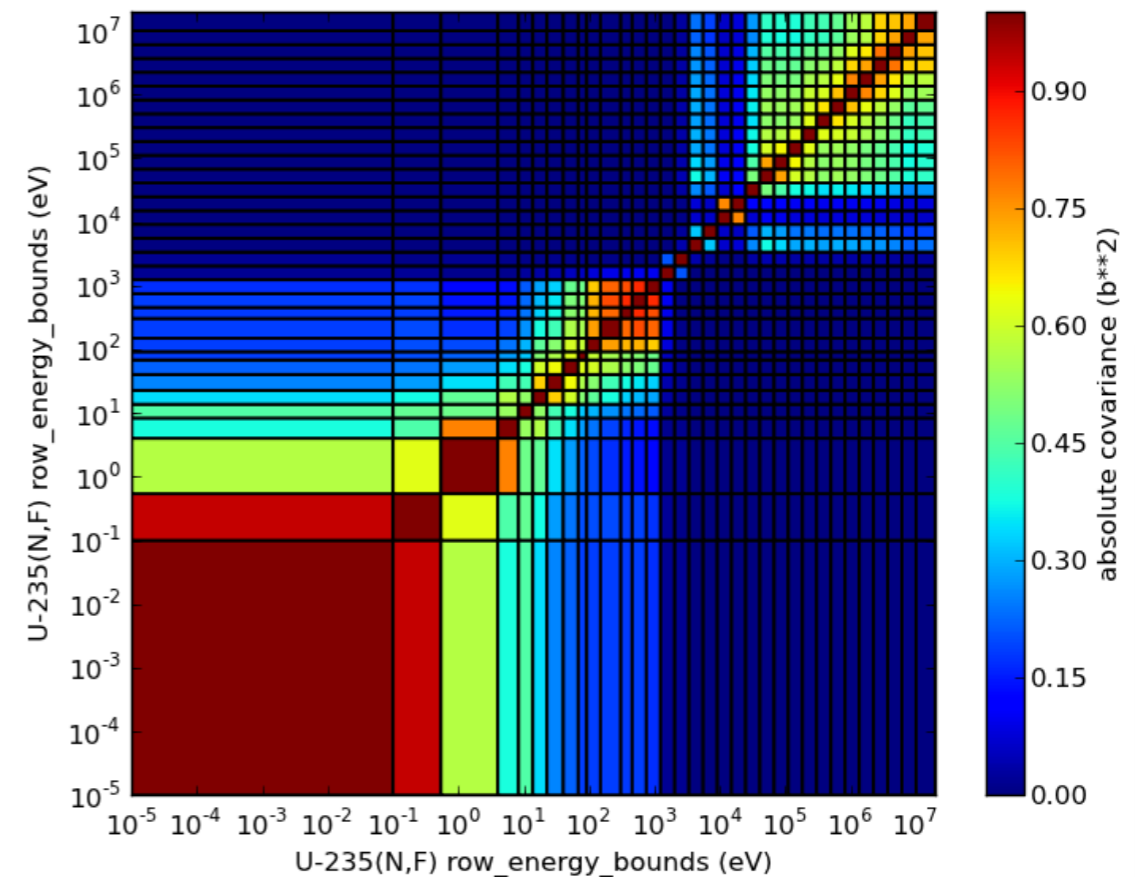
Only structural materials with MF34

^{23}Na , ^{55}Mn , ^{56}Fe , $^{180-186}\text{W}$

Developing cross-isotope cross-reaction covariances

- EXFOR contains many forms of coupled cross section data:
 - reaction combinations
 - reactions on elemental targets
 - non-elastic data
 - ratios to monitors (unused)
 - “isomers math” (unused)
- Goal: Refit all COMMARA-2.0 priority cross sections simultaneously
 - 33-group structure
 - ENDF/B-VII.1 covariances for prior
 - includes “on-diagonal” covariance
 - includes standards “off-diagonals”
 - use EXFOR entry’s REACTION string to construct linearized kernel

End product is covariance,
not mean values



Integral experiment uncertainty simulations using COMMARA 2 & 3.0 β

SFR

Isotope	σ_{cap}	σ_{fiss}	ν	σ_{el}	σ_{inel}	χ	P_1^{el}	Total
²⁴⁰ Pu	206	241	439	39	113	169	0	580
⁵⁶ Fe	195	0	0	320	321	0	88	501
²⁴⁵ Cm	8	444	50	0	3	0	0	447
²³⁸ U	50	9	33	51	230	0	0	244
²⁴⁴ Cm	147	141	111	2	5	0	0	232
²³⁸ Pu	117	119	154	8	32	28	0	231
²⁴² Pu	176	93	81	7	15	0	0	216
²³⁹ Pu	131	136	41	27	78	100	0	233
²⁴¹ Pu	95	45	34	2	38	0	0	117
²⁴³ Am	55	58	13	2	18	0	0	83
²³ Na	8	0	0	21	182	0	145	234
Total	423	567	492	329	459	198	170	1063

2.0

ADMAB

Isotope	σ_{cap}	σ_{fiss}	ν	σ_{el}	σ_{inel}	χ	P_1^{el}	Total
²⁴¹ Am	231	122	172	11	116	0	0	333
²⁴¹ Pu	73	42	33	0	57	0	0	107
²⁴⁵ Cm	13	1063	115	0	10	0	0	1070
²⁴⁴ Cm	505	651	516	1	37	0	0	973
²⁴³ Am	300	449	95	3	229	0	0	594
²³⁷ Np	183	171	65	1	97	0	0	277
²⁰⁹ Bi	64	0	0	66	166	0	0	190
²⁴⁰ Pu	56	88	151	8	64	82	0	211
¹⁵ N	1	0	0	216	13	0	0	216
²³⁹ Pu	91	126	35	15	118	129	0	237
^{242m} Am	11	142	10	0	4	0	0	143
²³⁸ Pu	32	56	59	2	19	3	0	89
Total	674	1360	593	227	355	153	0	1690

3.0 β

Isotope	σ_{cap}	σ_{fiss}	ν	σ_{el}	σ_{inel}	χ	P_1^{el}	Total
²⁴⁰ Pu	153	94	119	36	106	95	0	261
⁵⁶ Fe	194	0	0	320	320	0	80	499
²⁴⁵ Cm	39	46	67	0	2	21	0	93
²³⁸ U	48	9	33	52	231	45	0	248
²⁴⁴ Cm	94	31	90	1	3	3	0	133
²³⁸ Pu	63	26	56	5	8	30	0	93
²⁴² Pu	40	41	26	6	14	55	1	85
²³⁹ Pu	132	141	41	27	78	100	0	236
²⁴¹ Pu	95	70	34	2	38	0	0	128
²⁴³ Am	55	58	13	2	18	0	0	83
²³ Na	102	0	0	17	182	0	35	212
Total	345	206	186	327	456	160	87	738

Isotope	σ_{cap}	σ_{fiss}	ν	σ_{el}	σ_{inel}	χ	P_1^{el}	Total
²⁴¹ Am	230	122	172	11	116	0	0	333
²⁴¹ Pu	73	74	33	0	57	0	0	123
²⁴⁵ Cm	57	112	149	0	6	69	0	207
²⁴⁴ Cm	311	148	416	1	25	45	1	543
²⁴³ Am	300	449	95	3	229	0	0	594
²³⁷ Np	314	171	65	1	64	0	0	369
²⁰⁹ Bi	63	0	0	66	166	0	0	190
²⁴⁰ Pu	44	34	37	8	60	60	0	109
¹⁵ N	1	0	0	216	13	0	0	216
²³⁹ Pu	91	131	35	15	118	129	0	240
^{242m} Am	0	0	0	0	0	0	0	0
²³⁸ Pu	17	10	21	1	5	14	0	33
Total	602	551	492	227	346	165	1	1052

Summary

- ENDF/B-VII.1 released Dec 2011
- ENDF files processed with NJOY using AFCI 33-group structure, flux = thermal, 1/E, fission spec ($E > 100$ keV)
- Preliminary files tested using Sigma-QA and UnCor procedures for Quality Assurance
- Subset provided as COMMARA-3.0 β
- Final release dependent on funding

EXTRAS



Commara 2.0 materials

- 110 materials most relevant to fast reactor R&D
 - 12 light nuclei (LANL)
 - 78 structural materials (BNL)
 - 20 major and minor actinides (LANL + BNL)
- 135 files
 - 110 cross section covariances,
 - 20 nubars,
 - 3 PFNS,
 - 2 mubars

¹ H	²⁸ Si	⁹² Mo	²⁰⁸ Ag	¹⁴⁹ Sm	²³² Th
² H	²⁹ Si	⁹⁴ Mo	²¹⁷ I	¹⁵¹ Sm	²³³ U
⁴ He	³⁰ Si	⁹⁵ Mo	²²⁹ I	¹⁵² Sm	²³⁴ U
⁶ Li	⁵⁰ Cr	⁹⁶ Mo	¹³¹ Xe	¹⁵³ Eu	²³⁵ U
⁷ Li	⁵² Cr	⁹⁷ Mo	¹³² Xe	¹⁵⁵ Eu	²³⁶ U
⁹ Be	⁵³ Cr	⁹⁸ Mo	¹³⁴ Xe	¹⁵⁶ Gd	²³⁸ U
¹⁰ B	⁵⁵ Mn	¹⁰⁰ Mo	¹³³ Cs	¹⁵⁶ Gd	²³⁷ Np
¹¹ B	⁵⁴ Fe	⁹⁹ Tc	¹³⁵ Cs	¹⁵⁷ Gd	²³⁸ Pu
¹² C	⁵⁶ Fe	¹⁰¹ Ru	¹³⁹ La	¹⁵⁸ Gd	²³⁹ Pu
¹⁵ N	⁵⁷ Fe	¹⁰² Ru	¹⁴¹ Ce	¹⁶⁰ Gd	²⁴⁰ Pu
¹⁶ O	⁵⁸ Ni	¹⁰³ Ru	¹⁴³ Pr	¹⁶⁶ Er	²⁴¹ Pu
¹⁹ F	⁶⁰ Ni	¹⁰⁴ Ru	¹⁴³ Nd	¹⁶⁷ Er	²⁴² Pu
²³ Na	⁹⁰ Zr	¹⁰⁶ Ru	¹⁴⁵ Nd	¹⁶⁸ Er	²⁴¹ Am
²⁴ Mg	⁹¹ Zr	¹⁰³ Rh	¹⁴⁶ Nd	¹⁷⁰ Er	^{242m} Am
²⁵ Mg	⁹² Zr	¹⁰⁵ Pd	¹⁴⁸ Nd	²⁰⁸ Pb	²⁴³ Am
²⁶ Mg	⁹³ Zr	¹⁰⁶ Pd	¹⁴⁷ Pm	²⁰⁶ Pb	²⁴² Cm
²⁷ Al	⁹⁴ Zr	¹⁰⁷ Pd		²⁰⁷ Pb	²⁴³ Cm
	⁹⁵ Zr	¹⁰⁸ Pd		²⁰⁸ Pb	²⁴⁴ Cm
	⁹⁶ Zr			²⁰⁹ Bi	²⁴⁵ Cm
	⁹⁸ Nb				²⁴⁶ Cm

COMMARA-2 Methodology

- Covariance evaluation methodology determined by priorities:
 - Most important materials treated individually
 - Medium importance materials treated with simplified methods
 - Low priority materials (mostly fission products) treated with low-fidelity type approach

COMMARA-2 Methodology

Thermal and Resonance Region

- Source of data
- Experiments
- ENDF file (retroactive method)
- Atlas of Neutron Resonances (ANR)
- SAMMY analysis
- full analysis (MF32, Exp. data)
- retroactive (MF32, ENDF file)
- EMPIRE Resonance Module (MF32, ANR, scattering radius and thermal point uncertainties reproduced through correlations (if possible))
- “Kernel Approximation” (MF33, ANR)
- MF32 with systematic uncertainties in MF33
- ‘low-fidelity’ (Mark Williams) solution
- Assimilation

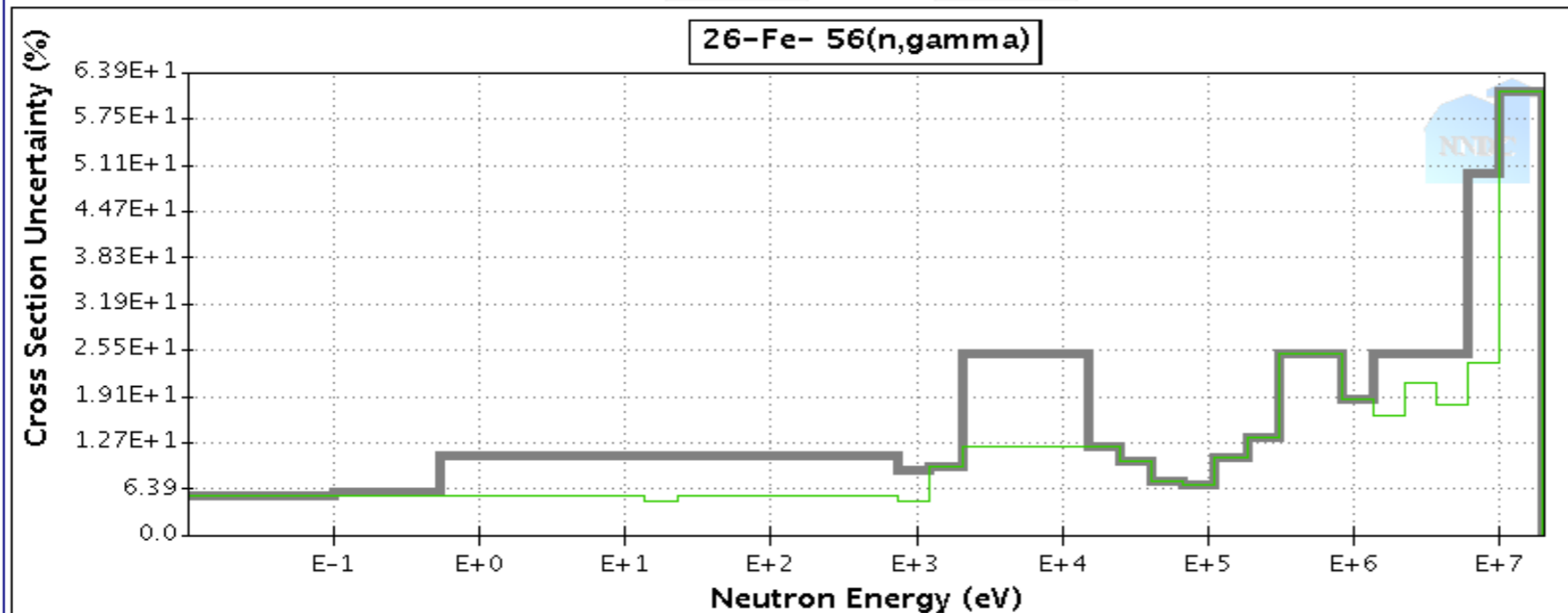
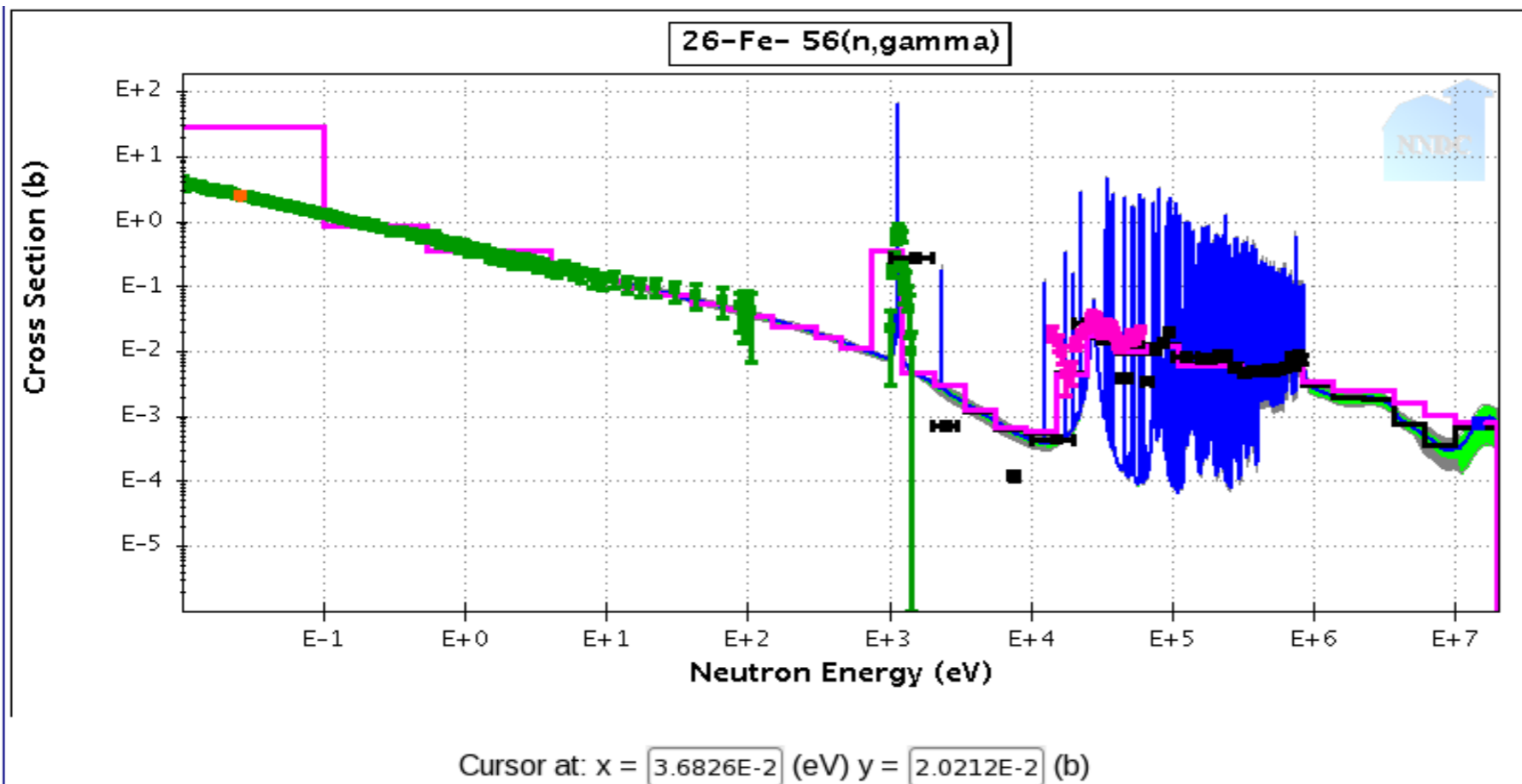
Fast neutron range (MF33)

- EMPIRE/KALMAN considering experimental data
- Least Square fitting of experimental data (SOK code)
- EMPIRE/KALMAN without experimental data (Low-Fidelity)
- Dispersion analysis - differences among evaluations (and exp. data)
- Reconsider previous work (ENDF/B-VI.8, Low-Fidelity)
- Visual analysis of experimental data
- Assimilation

^{60}Ni capture integral quantities from Sigma-QA

Library	Capture				
	THERMAL	RI 0.5 - 2E+7 eV	MACS 30 keV	^{252}Cf	14 MeV
ENDF/B-VII.0	2.772	1.412	2.826E-2	4.022E-3	2.859E-4
JEFF3.1	2.772	1.412	2.826E-2	6.033E-3	7.558E-4
JENDL4.0	2.913	1.472	2.792E-2	6.172E-3	6.992E-5
ROSFOND	2.772	1.412	2.826E-2	6.033E-3	7.558E-4
ENDF/B-VI.8	2.772	1.406	2.826E-2	4.022E-3	2.859E-4
CENDL3.1	2.772	1.413	2.826E-2	5.825E-3	1.131E-3
KADONIS			2.990E-2		
Atlas	2.500	1.400			
Kadonis Δ			7.000E-4 2.34%		
Atlas Δ	6.000E-2 2.40%	2.000E-1 1.42E+1%			
AFCI2.0 Δ	1.386E-1 5.00%	1.183E-1 8.37%	1.811E-3 6.40%	2.902E-4 7.21%	6.591E-5 2.30E+1%
Recommended Δ	1.430E-1 5.15%	2.017E-1 1.42E+1%	7.968E-4 2.81%		

Example of Sigma-QA plot



Update Plot Reset

$1E-2 \leq E_n$ (eV) $\leq 2E7$ Log

$1E-6 \leq \sigma$ (b) $\leq 1.924E2$ Log

- ENDF/B-VII.0 pointwise
- AFCI 1.2 uncertainty
- AFCI 1.3 uncertainty
- AFCI 2.0 uncertainty
- AFCI 2.0' uncertainty

Group cross sections with 1/E flux

- ENDF/B-VII.0 group
- JENDL-4.0 group
- JEFF-3.1 group
- CENDL-3.1 group
- ROSFOND group
- ENDF/B-VI.8 group

There are 7 EXFOR datasets

- Check/Uncheck All
- Huang Zheng-De 1980
- Shcherbakov 1977
- Shcherbakov 1977
- Allen 1982
- Allen 1976
- Macklin 1964
- Pomerance 1952

Remove EXFOR

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